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 SCHWENCER, A. Licensing Branch 2

SUBJECT: Responds to NRC 830608 telecopy requesting clarification of three areas of util preservice insp program. Clarification of preservice insp of scram discharge vol sys encl.

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 TITLE: Licensing Submittal: PSAR/FSAR Amdts & Related Correspondence.

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	NRR/DHFS/HFEB40	1 1	NRR/DHFS/LQB 32	1 1
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	NRR/DSI/AEB 26	1 1	NRR/DSI/ASB	1 1
	NRR/DSI/CPB 10	1 1	NRR/DSI/CSB 09	1 1
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	NRR/DSI/PSB 19	1 1	NRR/DSI/RAB 22	1 1
	NRR/DSI/RSB 23	1 1	REG FILE 04	1 1
	RGN5	3 3	RM/DDAMI/MIB	1 0
EXTERNAL:	ACRS 41	6 6	BNL (AMDTS ONLY)	1 1
	DMB/DSS (AMDTS)	1 1	FEMA-REP DIV 39	1 1
	LPDR 03	1 1	NRC PDR 02	1 1
	NSIC 05	1 1	NTIS	1 1

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## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

June 15, 1983  
G02-83-523

Docket No. 50-397

Director of Nuclear Reactor Regulation  
Attention: Mr. A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Schwencer:

Subject: NUCLEAR PROJECT NO. 2  
PSI SUMMARY REPORT CLARIFICATION

Reference: Telecopy, A. Schwencer (NRC) to R. Nelson (SS),  
6/8/83 "Washington Public Power Supply System,  
Washington Nuclear Project No. 2, Review of  
the Preservice Inspection Program"

The referenced telecopy requested clarification of three areas of the WNP-2 PSI program and required response by June 10, 1983. Accordingly, the attached clarification of the WNP-2 PSI program is provided.

Should you have any further questions, please contact Mr. R. M. Nelson, Manager, WNP-2 Licensing.

Very truly yours,



G. D. Bouchey  
Manager, Nuclear Safety and Regulatory Programs

PLP/tmh  
Enclosure

cc: R Auluck - NRC  
WS Chin - BPA  
A Toth - NRC Site  
M Humm - NRC

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1. Preservice Inspection of the Scram Discharge Volume (SDV) System

WNP-2 will select the NRC alternate program and will examine, preservice, 10% of the welds in the SDV system that are greater than 4 inches in diameter using a volumetric technique.

2. PSI Relief Requests

A. Repairs due to PSI

During the performance of the PSI examinations, it was discovered that some welds prepared for ISI by blending were below minimum wall thickness. The PSI examination contractor performed UT thickness measurements of all welds that had been blended for ISI. Since that time, the construction contractor has performed UT thickness measurements on all ISI welds after blending. From these two programs welds whose wall thickness has been reduced below minimum nominal thickness were identified and sent to the AE for evaluation.

The AE formally evaluated each weld which was less than 12 1/2% of the specified nominal wall thickness to the ASME Section III design criteria.\* All welds were either repaired by welding and reexamined or were accepted as is. Section XI, Appendix A criteria was not used to accept these welds for service. All welds "accepted as is" were compared to the original ASME Section III code requirements.

Two welds did not meet the pressure minimum design wall thickness when corrosion allowance was added and were not repaired. Both welds will be subject to an augmented ISI program to monitor the actual corrosion rate. The Supply System did not feel it was necessary to repair these welds because of the small (.007 and .010 inch) reduction in corrosion allowance. Even considering the reduced corrosion allowance, the calculated life expectancy for these two welds is 35 and 37 years. If the augmented ISI shows the actual corrosion rate is the same as or larger than specified rate, these welds will be reevaluated and repaired as necessary.

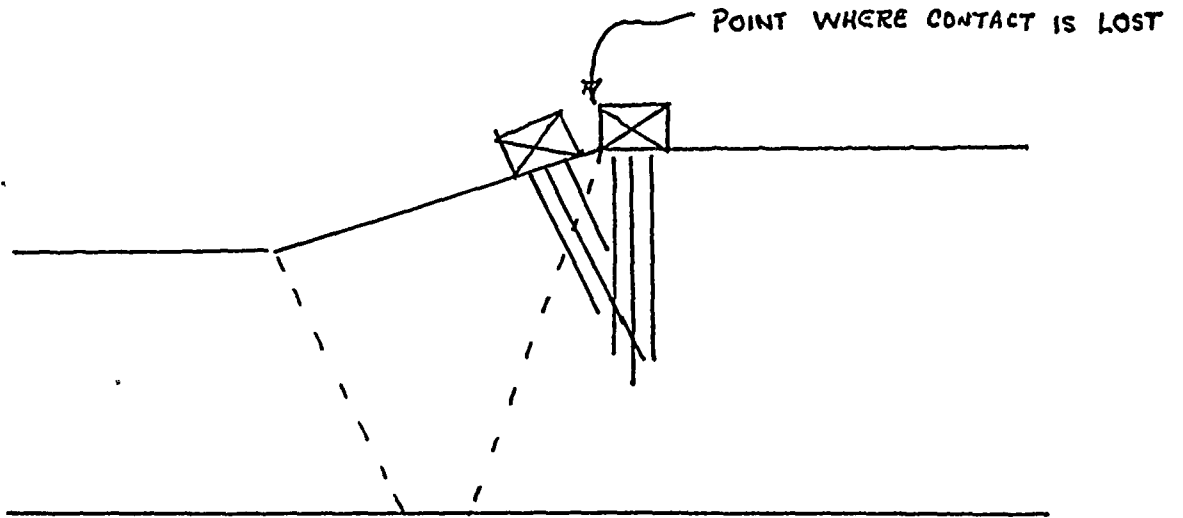
B. Mechanical PSI of the Reactor Vessel

As described in the WNP-2 PSI Summary Report, contact between the ultrasonic transducer and the reactor pressure vessel wall was maintained approximately 75% of the time on the transition from the first to second shell course. This statement should not be construed to mean that the volume of material specified to be examined by ASME Section XI was not examined. To the contrary, the Supply System feels that a full code examination was performed. NES, Inc. performed this examination manually as described on pages 3, 4 and 5 of the WNP-2 PSI Summary Report. The LMT examination demonstrated that the mechanized equipment would produce results which are essentially the same as a manual examination. Regardless of whether a mechanized or manual examination is performed, a 100% examination is still achievable. This is because the examination is performed from the outside diameter of the vessel and from both directions, above and below the weld. Because of the slope, as shown below, the coverage overlaps.

\*Note: Welds which are 87 1/2% of nominal wall thickness are unconditionally considered acceptable for use.



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The above illustration is for a  $0^{\circ}$  examination. For the  $45^{\circ}$  and  $60^{\circ}$  shear wave examinations, the transducer would not need to ride over this transition to gain full coverage of the required weld volume.



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