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SCE&G®
A SCANA COMPANY

September 27, 2017
RC-17-0134

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Subject: VIRGIL C. SUMMER NUCLEAR STATION, UNIT 1
DOCKET NO. 50-395
OPERATING LICENSE NO. NPF-12
LICENSE AMENDMENT REQUEST – LAR-15-01424
IMPLEMENTATION OF WCAP-15376-P-A, REVISION 1
RESPONSE TO REQUEST FOR SUPPLEMENTAL INFORMATION

Reference: 1. T. D. Gatlin, SCE&G, letter to Document Control Desk, NRC, "License Amendment Request – LAR-15-01424 Implementation of WCAP-15376-P-A, Revision 1 – 'Risk-Informed Assessment of the RTS and ESFAS Surveillance Test Intervals and Reactor Trip Breaker Test and Completion Times'," dated December 16, 2015 [ML15356A048]

2. S.A. Williams, NRC e-mail to B. Thompson, SCE&G, "V.C. Summer Nuclear Station, Unit 1 - TSTF-411 application TS page 3/4 3-37" Supplemental Information Needed to correct Typographical error, dated September 13, 2017

South Carolina Electric & Gas Company (SCE&G), acting for itself and as agent for South Carolina Public Service Authority pursuant to 10 CFR 50.90, submitted License Amendment Request per Reference 1 concerning the implementation of WCAP-15376-P-A, Revision 1. During NRC staff review of the Technical Specification (TS) pages provided with the application dated December 16, 2015 (Reference 1), NRC staff identified an administrative error in Table 4.3-2 on TS Page 3/4 3-37 (Reference 2). This submittal contains SCE&G's corrected TS page.

This letter does not contain any new regulatory commitments.

If there are any questions or if additional information is needed, please contact Bruce L. Thompson at (803) 931-5042.

I certify under penalty of perjury that the foregoing is correct and true.

9/27/17
Executed on

George A. Lippard
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BD/GAL/hk

Attachments:
Enclosure 1: Description of Technical Specification Administrative Error
Enclosure 2: Mark-Up Technical Specification Page Correcting the Administrative Error
Enclosure 3: Clean Copy of Technical Specification Page Correcting the Administrative Error

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PRSF (RC-17-0134)

VIRGIL C. SUMMER NUCLEAR STATION (VCSNS) UNIT 1
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ENCLOSURE 1

DESCRIPTION OF TECHNICAL SPECIFICATION ADMINISTRATIVE ERROR

1. Description of Typographical Error

During NRC staff review of the Technical Specification (TS) pages provided with the application dated December 16, 2015 (Reference 1), NRC staff identified an administrative error in Table 4.3-2 Engineered Safety Feature Actuation System Instrumentation Surveillance Requirements on TS Page 3/4 3-37.

The administrative error is a duplicate line item in Table 4.3-2 Functional Unit 4.d. "Steam Flow in Two Steam Lines -- High Coincident with T_{avg} --Low-Low". Only one line is required to specify the surveillance requirements for Functional Unit 4.d. This error has been in the VCSNS TSs since the issuance of the facility operating license on August 6, 1982.

This request is specific to deleting the duplicate line in Table 4.3-2 Functional Unit 4.d on TS Page 3/4 3-37.

2. Corrections to the Affected TS Pages

The Mark-Up of Technical Specification Page Correcting the Administrative Error is found in Enclosure 2. The Clean Copy of Technical Specification Page Correcting the Administrative Error is found in Enclosure 3.

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ENCLOSURE 2

MARK-UP TECHNICAL SPECIFICATION PAGE CORRECTING THE ADMINISTRATIVE ERROR

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	ACTUATION LOGIC TEST	MASTER RELAY TEST	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
4. STEAM LINE ISOLATION								
a. Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	R(3)	1, 2, 3
c. Reactor Building Pressure-High-2	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
d. Steam Flow in Two Steam Lines--High Coincident with T_{avg}--Low-Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
e. Steam Line Pressure Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
5. TURBINE TRIP AND FEEDWATER ISOLATION								
a. Steam Generator Water Level--High-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2
b. Automatic Actuation Logic and Actuation Relay	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	R(3)	1, 2
6. EMERGENCY FEEDWATER								
a. Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	R(3)	1, 2, 3
c. Steam Generator Water Level--Low-Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3

Remove duplicate line

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ENCLOSURE 3

CLEAN COPY OF TECHNICAL SPECIFICATION PAGE CORRECTING THE ADMINISTRATIVE ERROR

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	ACTUATION LOGIC TEST	MASTER RELAY TEST	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
4. STEAM LINE ISOLATION								
a. Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	Q(1)	Q(1)	R(3)	1, 2, 3
c. Reactor Building Pressure-High-2	S	R	SA	N.A.	N.A.	N.A.	N.A.	1, 2, 3
d. Steam Flow in Two Steam Lines--High Coincident with T_{avg}--Low-Low	S	R	SA	N.A.	N.A.	N.A.	N.A.	1, 2, 3
e. Steam Line Pressure Low	S	R	SA	N.A.	N.A.	N.A.	N.A.	1, 2, 3
5. TURBINE TRIP AND FEEDWATER ISOLATION								
a. Steam Generator Water Level--High-High	S	R	SA	N.A.	N.A.	N.A.	N.A.	1, 2
b. Automatic Actuation Logic and Actuation Relay	N.A.	N.A.	N.A.	N.A.	Q(1)	Q(1)	R(3)	1, 2
6. EMERGENCY FEEDWATER								
a. Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	Q(1)	Q(1)	R(3)	1, 2, 3
c. Steam Generator Water Level--Low-Low	S	R	SA	N.A.	N.A.	N.A.	N.A.	1, 2, 3