

CATEGORY 10

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9907260052 DOC.DATE: 99/07/19 NOTARIZED: NO DOCKET #
 FACIL:50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH.NAME AUTHOR AFFILIATION
 TEED,R.C. Rochester Gas & Electric Corp.
 MECREDY,R.C. Rochester Gas & Electric Corp.
 RECIP.NAME RECIPIENT AFFILIATION

VISSING,G.S.

SUBJECT: LER 99-S01-00:on 990617,determined that temporary unescorted access had been granted to contractor employee.Caused by incomplete info re circumstances of individual military separation.Individual access was revoked.With 990719 ltr.

DISTRIBUTION CODE: IE74T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 3
 TITLE: Safeguards Phys Sec Event Pt. 73.71 (Public Available)

NOTES:License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	VISSING,G.	1 0		
INTERNAL	FILE CENTER	1 1	NMSS/FCSS/FCOB	1 1
	NUDOCS-FULLTEXT	1 1	RES/DRAA/OERAB	1 1
	RGN1 01	1 1		
EXTERNAL:	NRC PDR	1 1		

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE. TO HAVE YOUR NAME OR ORGANIZATION REMOVED FROM DISTRIBUTION LISTS OR REDUCE THE NUMBER OF COPIES RECEIVED BY YOU OR YOUR ORGANIZATION, CONTACT THE DOCUMENT CONTROL DESK (DCD) ON EXTENSION 415-2083

FULL TEXT CONVERSION REQUIRED
 TOTAL NUMBER OF COPIES REQUIRED: LTTR 7 ENCL 6

C
A
T
E
G
O
R
Y

1

D
O
C
U
M
E
N
T



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001



AREA CODE 716 546-2700

ROBERT C. MECREDY
Vice President
Nuclear Operations

July 19, 1999

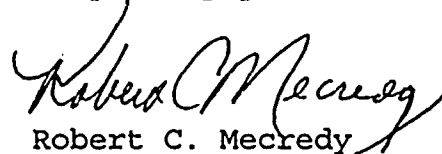
U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: LER 1999-S01, Safeguards Event
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

The attached Licensee Event Report LER 1999-S01 is submitted in accordance with 10 CFR 73.71 and Section I(b) of Appendix G, Reportable Safeguards Events.

Very truly yours,


Robert C. Mecredy

xc: Mr. Guy S. Vissing (Mail Stop 8C2)
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

U.S. NRC Ginna Senior Resident Inspector

//

IE 74

9907260052 990719
PDR ADCK 05000244
S PDR

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)

FACILITY NAME (1)

R. E. Ginna Nuclear Power Plant

DOCKET NUMBER (2)

05000244

PAGE (3)

1 OF 3

TITLE (4)

Safeguards Event

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
06	17	1999	1999	-- S01	-- 00	07	19	1999		05000
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)							
1			20.2201(b)			20.2203(a)(2)(v)			50.73(a)(2)(i)(B)	50.73(a)(2)(viii)
POWER LEVEL (10)			20.2203(a)(1)			20.2203(a)(3)(i)			50.73(a)(2)(ii)	50.73(a)(2)(x)
100			20.2203(a)(2)(i)			20.2203(a)(3)(ii)			50.73(a)(2)(iii)	X 73.71
			20.2203(a)(2)(ii)			20.2203(a)(4)			50.73(a)(2)(iv)	OTHER
			20.2203(a)(2)(iii)			50.36(c)(1)			50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 368A
			20.2203(a)(2)(iv)			50.36(c)(2)			50.73(a)(2)(vii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

Ronald C. Teed - Supervisor Nuclear Security

TELEPHONE NUMBER (Include Area Code)

(716) 771-3232

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE).

NO

X

EXPECTED
SUBMISSION
DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On June 17, 1999, at 0826 hours, a nuclear access authorization event was identified and determined to be reportable to the Nuclear Regulatory Commission in accordance with 10CFR73.71, Appendix G Section I(b). It was determined that temporary unescorted access had been granted to a contractor employee who would not have been granted access had complete information concerning the circumstances of his military separation been known.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
R.E. Ginna Nuclear Power Plant	05000244	1999	-- S01	-- 00	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. Pre-Event Plant Conditions

The unit was in Mode 1, at normal operations at approximately 100% steady-state reactor power.

II. Description of Event

On April 29, 1999, the Ginna Station Access Authorization office received the requested military records for an individual who had been granted temporary unescorted access on February 18, 1999. After review of this documentation, it was determined that the individual's trustworthiness and reliability could be called into question. Unescorted access was suspended immediately (on April 29) pending completion of an investigation. Upon completion of this investigation, unescorted access was revoked on June 17, 1999.

III. Event Summary

While reviewing the Department of Defense form DD-214 supplied by this individual during in-processing for unescorted access to Ginna Station in February 1999, it was noted that he had an honorable discharge from the U.S. Navy. The narrative reason for separation was "Personality Disorder". As part of the background investigation process the individual was required to have a clinical interview with the Rochester Gas and Electric (RG&E) staff psychologist. During this interview the information regarding his Navy discharge was discussed, along with other personal information. Based on the information obtained at the time, the psychologist recommended the individual for unescorted access. The rest of the individual's background investigation did not reveal any other information that would indicate the individual to be untrustworthy or unreliable. Per NRC Regulatory Guide 5.66, it is not required to do any additional investigation into a military discharge that is honorable.

Information was received from the National Personnel Records Center (NPRC) on April 29, 1999. Attached to the DD-214 received on April 29 was additional information pertaining to the individual's psychological evaluation conducted while he was in the Navy. The RG&E staff psychologist was provided with this additional information. His recommendation was to attempt to contact the Navy to obtain more detailed information. The individual's access authorization was suspended (on April 29), pending receipt of this information. When RG&E was unable to obtain additional information from the Navy, the psychologist recommended a complete independent psychological evaluation.

On June 3 and June 10, 1999, the individual was evaluated and tested by an independent psychologist. This report was then forwarded to the RG&E staff psychologist. Upon review of the report on June 17, 1999, it was recommended that the individual's unescorted access authorization be revoked.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
R.E. Ginna Nuclear Power Plant	05000244	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
		1999	-- S01	-- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

IV. Corrective Action

Short Term

Short term corrective action was accomplished by suspending the individual's unescorted access to Ginna Station immediately upon receipt of the military documentation on April 29, 1999, and revoking this access authorization on June 17, 1999.

Long Term

The individual's supervisors were interviewed regarding the individual's activities during the period when he had been granted unescorted access. No unusual behavior was noted.

All components of the process for granting the individual unescorted access were reviewed and determined to be in accordance with the requirements of 10CFR73.56 and NRC Regulatory Guide 5.66.