

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, NY 14649

March 10, 1999

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING THE R.E.  
GINNA NUCLEAR POWER PLANT, SUMMARY REPORT ON THE  
VERIFICATION OF SEISMIC ADEQUACY OF MECHANICAL AND ELECTRICAL  
EQUIPMENT IN OPERATING REACTORS, DATED JANUARY 31, 1997  
(TAC NO. M69449)

Dear Dr. Mecredy:

The NRC staff has completed its review of your submittal on verification of seismic adequacy of mechanical and electrical equipment in operating reactors, dated May 27, 1997. The staff requires additional information regarding operator actions specified in the report in order to make a safety determination. The enclosed request for additional information identifies the information needed to continue the review. A draft of the needed information was e-mailed to your staff on February 12, 1999, for the purpose of making a determination as to when you would be able to respond. We discussed this with your staff on February 23, 1999, at which time your staff indicated that, considering the upcoming refueling outage and other activities, you would be able to respond within 90 days of the date of this letter. If you have any questions concerning this request, please advise me.

Sincerely,

Original signed by:  
Guy S. Vissing, Senior Project Manager  
Project Directorate I-1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

1/1  
d-61

9903190020 990310  
PDR ADOCK 05000244  
P PDR

Docket No. 50-244

Enclosure: As stated

**NRC FILE CENTER COPY**

cc w/encl: See next page

**DISTRIBUTION:**

PUBLIC

PDI-1 R/F

J. Zwolinski/S. Black

S. Bajwa

S. Little

G. Vissing

R. Blough, RI

OGC

ACRS

R. Eckenrode

G. Galletti

Docket File

DOCUMENT NAME: G:\GINNA\RAI69449.WPD

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	PH:PDI-1	E	LA:PDI-1	D:PDI-1			
NAME	GVissing/rsf	3/10/99	SLittle	SBajwa			
DATE	03/10/99		03/10/99	03/10/99			

170071

Official Record Copy



REQUEST FOR ADDITIONAL INFORMATION REGARDING THE LICENSEE'S ANALYSIS

ON THE VERIFICATION OF SEISMIC ADEQUACY OF MECHANICAL AND

ELECTRICAL EQUIPMENT IN OPERATING REACTORS

DATED JANUARY 31, 1997, R.E. GINNA NUCLEAR POWER PLANT

DOCKET NUMBER 50-244

The NRC staff has completed its review of the licensee's submittal dated January 31, 1997. The staff requires additional information regarding operator actions specified in the report in order to make a safety determination.

- a. Describe what reviews were performed to determine if any local operator actions required to safely shutdown the reactor (i.e., implement the Safe Shutdown Equipment List (SSEL)) could be affected by potentially adverse environmental conditions (such as loss of lighting, excessive heat or humidity, or in-plant barriers) resulting from the seismic event. Describe how staffing was evaluated and describe the reviews which were conducted to ensure operators had adequate time and resources to respond to such events.
- b. As part of the licensee's review, were any control room structures which could impact the operator's ability to respond to the seismic event identified? Such items might include but are not limited to: MCR ceiling tiles, non-bolted cabinets, and non-restrained pieces of equipment (i.e., computer keyboards, monitors, stands, printers, etc.). Describe how each of these potential sources of interactions has been evaluated and describe the schedule for implementation of the final resolutions.
- c. Describe what reviews were performed to determine if any local operator actions were required to reposition "bad actor relays." For any such activities describe how adverse environmental conditions (such as loss of lighting, excessive heat or humidity, or in-plant barriers) resulting from the seismic event were analyzed and dispositioned. Describe how staffing was evaluated and describe the reviews which were conducted to ensure operators had adequate time and resources to respond to such events.
- d. Describe which of the operator actions associated with resetting SSEL equipment affected by postulated relay chatter are considered to be routine and consistent with the skill of the craft. If not considered skill of the craft, what training and operational aids were developed to ensure the operators will perform the actions required to reset affected equipment?
- e. Assume the alarms associated with "bad actor relays" are expected to annunciate during the seismic event. Do the operators have to respond to those annunciators and review the annunciator response procedures associated with them for potential action? How would those additional actions impact the operators ability to implement the Normal, Abnormal, and Emergency Operating Procedures required to place the reactor in a safe shutdown condition?

Enclosure



- f. To the extent that Normal, Abnormal, and Emergency Operating Procedures were modified to provide plant staff with additional guidance on mitigating the A-46 Seismic Event, describe what training was required and provided to the licensed operators, non-licensed operators, and other plant staff required to respond to such events.

Dr. Robert C. Mecredy  
Rochester Gas and Electric Company

R.E. Ginna Nuclear Power Plant

cc:

Peter D. Drysdale, Sr. Resident Inspector  
R.E. Ginna Plant  
U.S. Nuclear Regulatory Commission  
1503 Lake Road  
Ontario, NY 14519

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. F. William Valentino, President  
New York State Energy, Research,  
and Development Authority  
Corporate Plaza West  
286 Washington Avenue Extension  
Albany, NY 12203-6399

Charles Donaldson, Esquire  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, NY 10271

Nicholas S. Reynolds  
Winston & Strawn  
1400 S Street N.W.  
Washington, DC 20005-3502

Ms. Thelma Wideman, Director  
Wayne County Emergency Management  
Office  
Wayne County Emergency Operations Center  
7336 Route 31  
Lyons, NY 14489

Ms. Mary Louise Meisenzahl  
Administrator, Monroe County  
Office of Emergency Preparedness  
111 West Falls Road, Room 11  
Rochester, NY 14620

Mr. Paul Eddy  
New York State Department of  
Public Service  
3 Empire State Plaza, 10th Floor  
Albany, NY 12223

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, NY 14649

March 10, 1999

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING THE R.E.  
GINNA NUCLEAR POWER PLANT, SUMMARY REPORT ON THE  
VERIFICATION OF SEISMIC ADEQUACY OF MECHANICAL AND ELECTRICAL  
EQUIPMENT IN OPERATING REACTORS, DATED JANUARY 31, 1997  
(TAC NO. M69449)

Dear Dr. Mecredy:

The NRC staff has completed its review of your submittal on verification of seismic adequacy of mechanical and electrical equipment in operating reactors, dated May 27, 1997. The staff requires additional information regarding operator actions specified in the report in order to make a safety determination. The enclosed request for additional information identifies the information needed to continue the review. A draft of the needed information was e-mailed to your staff on February 12, 1999, for the purpose of making a determination as to when you would be able to respond. We discussed this with your staff on February 23, 1999, at which time your staff indicated that, considering the upcoming refueling outage and other activities, you would be able to respond within 90 days of the date of this letter. If you have any questions concerning this request, please advise me.

Sincerely,

Original signed by:  
Guy S. Vissing, Senior Project Manager  
Project Directorate I-1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: As stated

cc w/encl: See next page

DISTRIBUTION:

PUBLIC	S. Bajwa	R. Blough, RI	R. Eckenrode
PDI-1 R/F	S. Little	OGC	G. Galletti
J. Zwolinski/S. Black	G. Vissing	ACRS	

DOCUMENT NAME: G:\GINNA\RAI69449.WPD

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	PH:PDI-1	E	LA:PDI-1	D:PDI-1				
NAME	GVissing/rsf	3/10/99	SLittle	SBajwa	8/12			
DATE	03/10/99		03/10/99	03/10/99				

Official Record Copy

44-03-19-0020 2pp







UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 10, 1999

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, NY 14649

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING THE R.E.  
GINNA NUCLEAR POWER PLANT, SUMMARY REPORT ON THE  
VERIFICATION OF SEISMIC ADEQUACY OF MECHANICAL AND ELECTRICAL  
EQUIPMENT IN OPERATING REACTORS, DATED JANUARY 31, 1997  
(TAC NO. M69449)

Dear Dr. Mecredy:

The NRC staff has completed its review of your submittal on verification of seismic adequacy of mechanical and electrical equipment in operating reactors, dated May 27, 1997. The staff requires additional information regarding operator actions specified in the report in order to make a safety determination. The enclosed request for additional information identifies the information needed to continue the review. A draft of the needed information was e-mailed to your staff on February 12, 1999, for the purpose of making a determination as to when you would be able to respond. We discussed this with your staff on February 23, 1999, at which time your staff indicated that, considering the upcoming refueling outage and other activities, you would be able to respond within 90 days of the date of this letter. If you have any questions concerning this request, please advise me.

Sincerely,

A handwritten signature in black ink, appearing to read "Guy S. Vissing", is written over the typed name.

Guy S. Vissing, Senior Project Manager  
Project Directorate I-1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: As stated

cc w/encl: See next page

99