

50-214



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 25, 1999

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: STAFF REVIEW OF THE SUBMITTAL BY ROCHESTER GAS AND ELECTRIC COMPANY TO APPLY LEAK-BEFORE-BREAK STATUS TO PORTIONS OF THE R.E. GINNA NUCLEAR POWER PLANT RESIDUAL HEAT REMOVAL SYSTEM PIPING (TAC NO. MA0389)

Dear Dr. Mecredy:

By letter dated November 11, 1997, you submitted a request for NRC review and approval of your leak-before-break (LBB) evaluation concerning portions of the R.E. Ginna Nuclear Power Plant residual heat removal (RHR) system piping. Your submittal was made in accordance with the provisions of Title 10 of the Code of Federal Regulations Part 50, Appendix A, General Design Criteria 4, which permits licensees to exclude the dynamic effects associated with postulated pipe ruptures from the facility's licensing basis if "analyses reviewed and approved by the Commission demonstrate that the probability of fluid system piping rupture is extremely low under conditions consistent with the design basis for the piping." LBB evaluations utilizing the guidance of NUREG-1061, Volume 3 and/or Draft Standard Review Plan Section 3.6.3 have been previously approved by the staff as a method for making such a demonstration.

The NRC staff has completed its evaluation of your submittal. The information provided by you in your original submittal, your August 6, 1998 response to the staff's request for additional information, and further information on the Ginna leakage detection system was sufficient to permit the staff to independently evaluate your conclusions. While the results of the staff's evaluation do differ with yours, the staff agrees with your conclusion that LBB behavior has been demonstrated for the analyzed portions of the RHR system piping. It should be noted that the staff's conclusion was predicated on your demonstration that the R. E. Ginna leakage detection system inside containment was capable of reliably detecting 0.25 gallons per minute of leakage. Therefore, the staff finds that you may remove consideration of the dynamic

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R. C. Mecredy

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effects associated with the postulated rupture of the analyzed portions of the RHR system piping from the licensing basis of R. E. Ginna Nuclear Power Plant.

The Safety Evaluation which addresses the technical basis for the staff's finding is enclosed. This completes our efforts for TAC No. MA0389.

Sincerely,



Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: Safety Evaluation

cc w/encl: See next page

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ORIGINAL SIGNED BY:

Guy S. Vissing, Senior Project Manager
Project Directorate I-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: Safety Evaluation

cc w/encl: See next page

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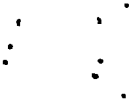
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DATE	2/24/99	2/24/99	2/25/99

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Dr. Robert C. Mecredy
Rochester Gas and Electric Company

R.E. Ginna Nuclear Power Plant

cc:

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