



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.O. 20555-0001

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.72  
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated March 31, 1997, as supplemented June 18, 1997, October 10, 1997, October 20, 1998, November 11, 1997, December 22, 1997, January 15, 1998, January 27, 1998, March 30, 1998, April 23, 1998, April 27, 1998, May 8, 1998 and May 22, 1998 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:



(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.72 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 30, 1998



ATTACHMENT TO LICENSE AMENDMENT NO. 72

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3.7-27  
3.7-28  
3.7-29  
3.7-30  
3.7-31

4.0-2  
4.0-3

Insert

3.7-27  
3.7-28  
3.7-29  
3.7-30  
3.7-31  
3.7-31a  
4.0-2  
4.0-3



### 3.7 PLANT SYSTEMS

#### 3.7.12 Spent Fuel Pool (SFP) Boron Concentration

| LCO 3.7.12      The SFP boron concentration shall be  $\geq 2300$  ppm.

| APPLICABILITY:    Whenever any fuel assembly is stored in the SFP.

#### ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. SFP boron concentration not within limit.	-----NOTE----- LCO 3.0.3 is not applicable. -----	
	A.1      Suspend movement of fuel assemblies in the SFP.	Immediately
	<u>AND</u> A.2      Initiate action to restore SFP boron concentration to within limit.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.12.1    Verify the SFP pool boron concentration is within limit.	7 days





## 3.7 PLANT SYSTEMS

## 3.7.13 Spent Fuel Pool (SFP) Storage

LCO 3.7.13 Fuel assembly storage in the spent fuel pool shall be maintained as follows:

- a. Fuel assemblies in Region 1 shall have a K-infinity of  $\leq 1.458$  and shall have initial enrichment and burnup within the acceptable area of Figure 3.7.13-1; and
- b. Fuel assemblies in Region 2 shall have initial enrichment and burnup within the acceptable area of the Figure 3.7.13-2.

APPLICABILITY: Whenever any fuel assembly is stored in the spent fuel pool.

## ACTIONS

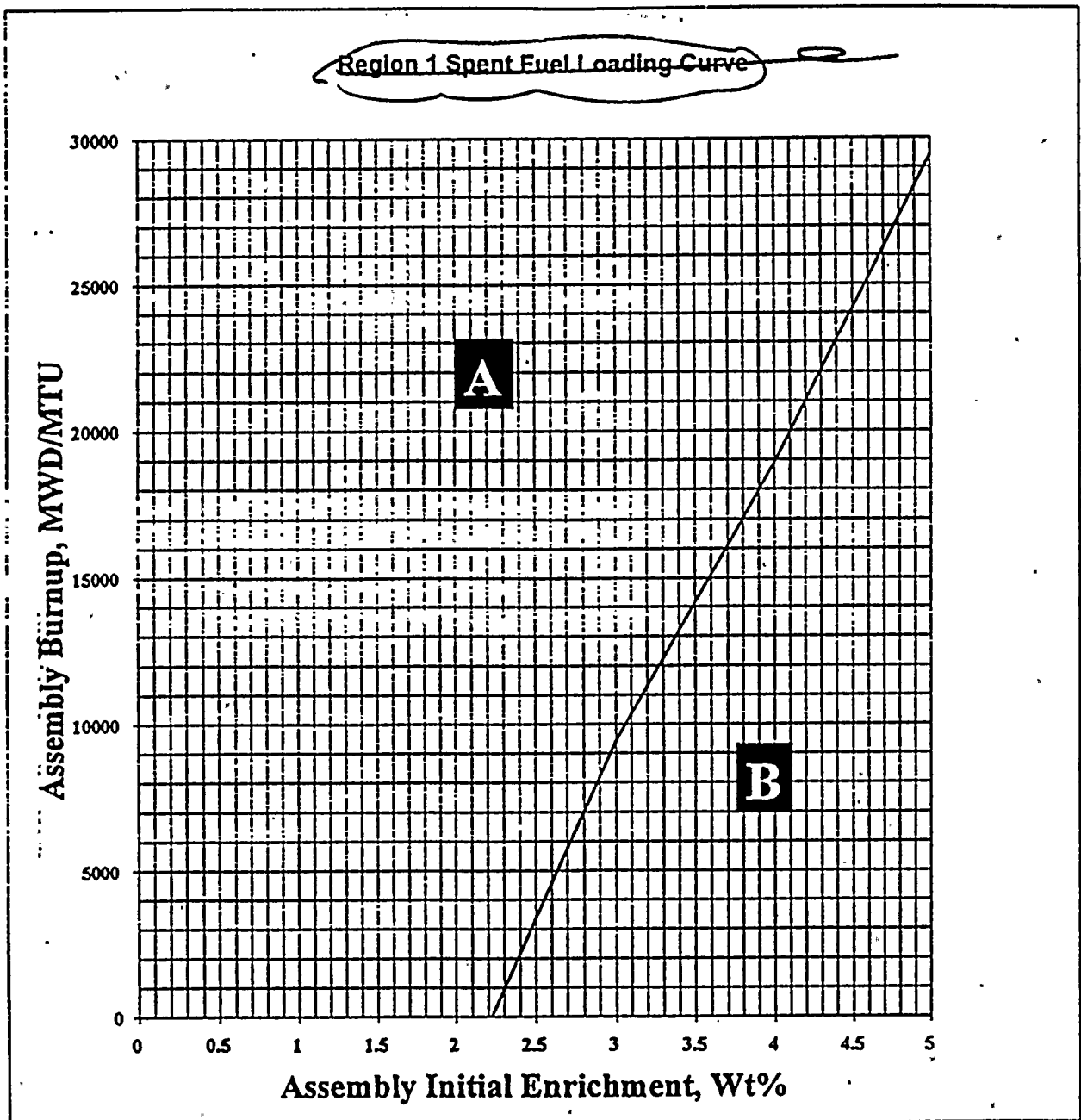
CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Requirements of the LCO not met for either region.	<p>A.1</p> <p>-----NOTE----- LCO 3.0.3 is not applicable. -----</p> <p>Initiate action to move the noncomplying fuel assembly to an acceptable storage location.</p>	Immediately



SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.13.1	Verify by administrative means the K-infinity of the fuel assembly is $\leq 1.458$ and that the initial enrichment and burnup is in accordance with Figure 3.7.13-1.	Prior to storing the fuel assembly in Region 1
SR 3.7.13.2	Verify by administrative means the initial enrichment and burnup of the fuel assembly is in accordance with Figure 3.7.13-2.	Prior to storing the fuel assembly in Region 2

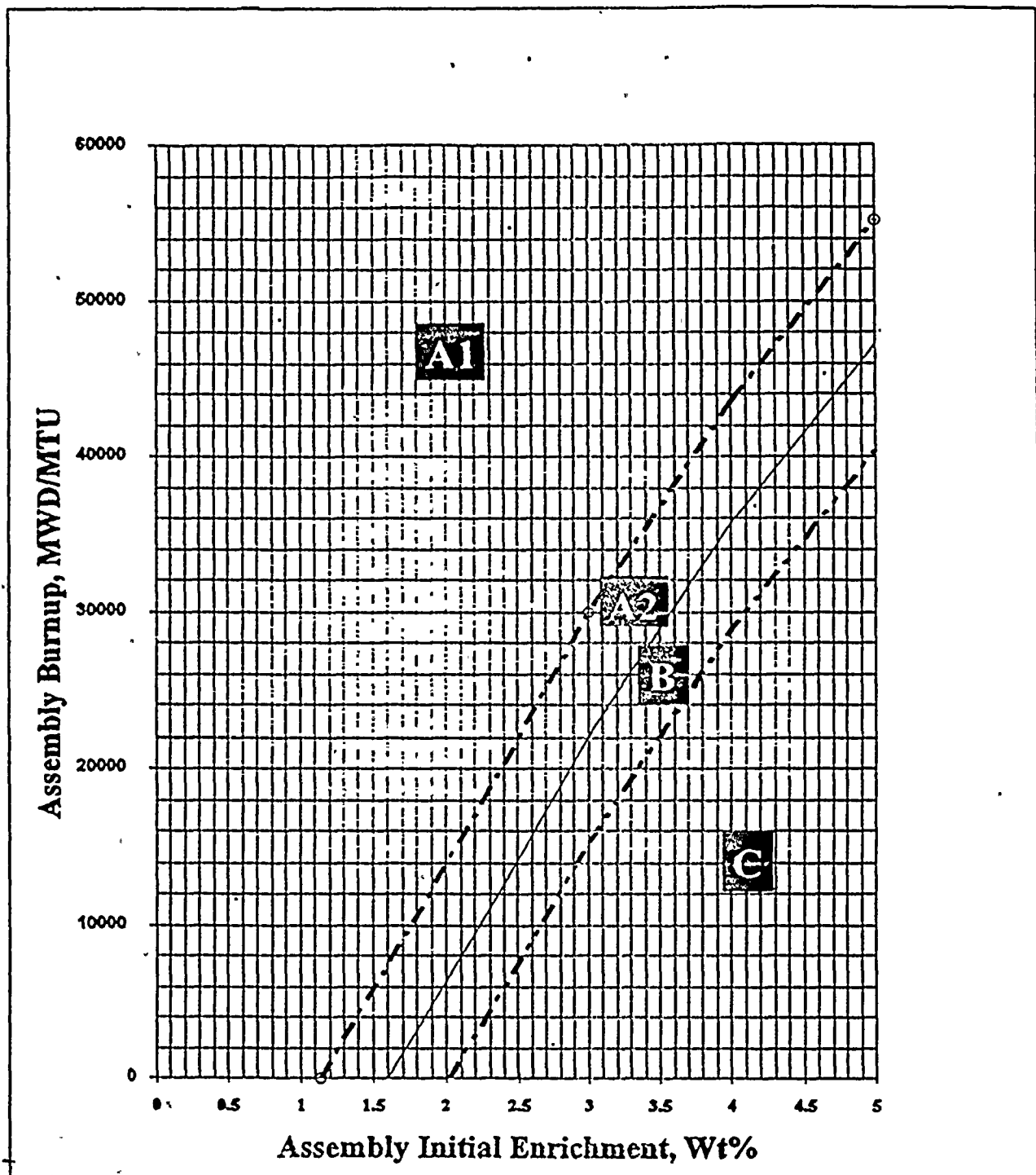




- A - Acceptable burnup domain for storage in any location within Region 1.  
B - Acceptable burnup domain for storage in cells with lead-in funnels only.

Figure 3.7.13-1  
Fuel Assembly Burnup Limits in Region 1





- A1 - Acceptable burnup domain for storage in any location within Region 2.
- A2 - Acceptable burnup domain for storage face-adjacent to a Type A1 or A2 assembly, or a water cell.
- B - Assembly burnup domain for storage face-adjacent to a Type A1 assembly or a water cell.
- C - Acceptable burnup domain for storage face-adjacent to a water cell only.

Figure 3.7.13-2  
Fuel Assembly Burnup Limits in Region 2



## 4.0 DESIGN FEATURES

---

### 4.2 Reactor Core (continued)

#### 4.2.2 Control Rod Assemblies

The reactor core shall contain 29 control rod assemblies. The control material shall be silver indium cadmium.

### 4.3 Fuel Storage

#### 4.3.1 Criticality

4.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

- a. Fuel assemblies having a maximum U-235 enrichment of 5.05 weight percent;
- b.  $k_{\text{eff}} \leq 0.95$  if fully flooded with unborated water\*, which includes an allowance for uncertainties as described in Section 9.1 of the UFSAR;
- c. Consolidated rod storage canisters may be stored in the spent fuel storage racks provided that the fuel assemblies from which the rods were removed meet all the requirements of LCO 3.7.13 for the region in which the canister is to be stored. The average decay heat of the fuel assembly from which the rods were removed for all consolidated fuel assemblies must also be  $\leq 2150$  BTU/hr.

4.3.1.2 The new fuel storage dry racks are designed and shall be maintained with:

- a. Fuel assemblies having a maximum U-235 enrichment of 5.05 weight percent;
- b.  $k_{\text{eff}} \leq 0.95$  if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.1 of the UFSAR; and
- c.  $k_{\text{eff}} \leq 0.98$  if moderated by aqueous foam, which includes an allowance for uncertainties as described in Section 9.1 of the UFSAR.

\* Until December 31, 1999, the spent fuel storage racks shall be maintained with a  $k_{\text{eff}} \leq 0.95$  when flooded with water containing  $\geq 2300$  ppm soluble boron

(continued)



#### 4.0 DESIGN FEATURES (continued)

---

#### 4.3 Fuel Storage (continued)

##### 4.3.2 Drainage

The spent fuel pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 257'0" (mean sea level).

##### 4.3.3 Capacity

The spent fuel pool is designed and shall be maintained with a storage capacity limited to no more than 1879 fuel assemblies and 1369 storage locations.

---