



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-18
ROCHESTER GAS AND ELECTRIC CORPORATION
R. E. GINNA NUCLEAR POWER PLANT
DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated September 29, 1997, as supplemented October 8, 1997, the Rochester Gas and Electric Corporation (the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise the Administrative Controls Section 5.6.6 of the Ginna TSs to incorporate a reference to the methodology for determining pressure/temperature (P/T) and low-temperature overpressure protection (LTOP) limits and correct a typographical error. The September 29, 1997, and October 8, 1997, submittals superseded in their entirety the requests for amendment dated December 13, 1996, as supplemented April 24, 1997, and June 3, 1997.

2.0 EVALUATION

The proposed amendment follows the guidance given in Generic Letter 96-03 for relocating LTOP and reactor coolant system (RCS) P/T limits to the RCS Pressure and Temperature Limits Report (PTLR). The licensee generated plant-specific P/T and LTOP limits using the NRC-approved WCAP-14040 methodology. The licensee has revised the PTLR curves, setpoints, values and parameters using portions of WCAP-14040, updated data and estimated values for neutron fluence. The staff has reviewed the PTLR and supporting submittals submitted by the licensee and by letter dated November 28, 1997, have found the report acceptable for referencing in the administrative controls section of the TSs in accordance with Generic Letter 96-03 and, therefore, have found the proposed TS change acceptable.

The proposed amendment corrects a minor spelling error in the TSs. We have determined this to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 59921). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: December 9, 1997

