

50-244



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 28, 1997

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, NY 14649

SUBJECT: R. E. GINNA - ACCEPTANCE FOR REFERENCING OF PRESSURE TEMPERATURE  
LIMITS REPORT, REVISION 2 (TAC NO. M96529)

Dear Dr. Mecredy:

- REFERENCES:
1. Amendment No. 48 to Facility Operating License No. DPR-18, R. E. Ginna Nuclear Power Plant, March 6, 1992.
  2. Letter from C. I. Grimes, NRC, To R. A. Newton, Westinghouse Electric Corporation, "Acceptance for Referencing of Topical Report WCAP-14040, Revision 1, 'Methodology Used to Develop Cold Overpressure Mitigating System Setpoints and RCS Heatup and Cooldown Limit Curves,'" October 16, 1995. (Also known as WCAP-14040-NP-A).
  3. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation (RG&E), to Document Control Desk, NRC, Attention A. R. Johnson, "Application for Amendment to Facility Operating License Methodology for Low Temperature Overpressure Protection (LTOP) Limits," February 9, 1996.
  4. Letter from A. R. Johnson, NRC, to R. C. Mecredy, Rochester Gas and Electric Corporation (RG&E), "R. E. Ginna Nuclear Power Plant - Pressurized Thermal Shock Evaluation (TAC No. M93827)," March 22, 1996.
  5. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation (RG&E), to Document Control Desk, NRC, Attention A. R. Johnson, "Application for Amendment to Facility Operating License Revision to Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)," April 22, 1996.

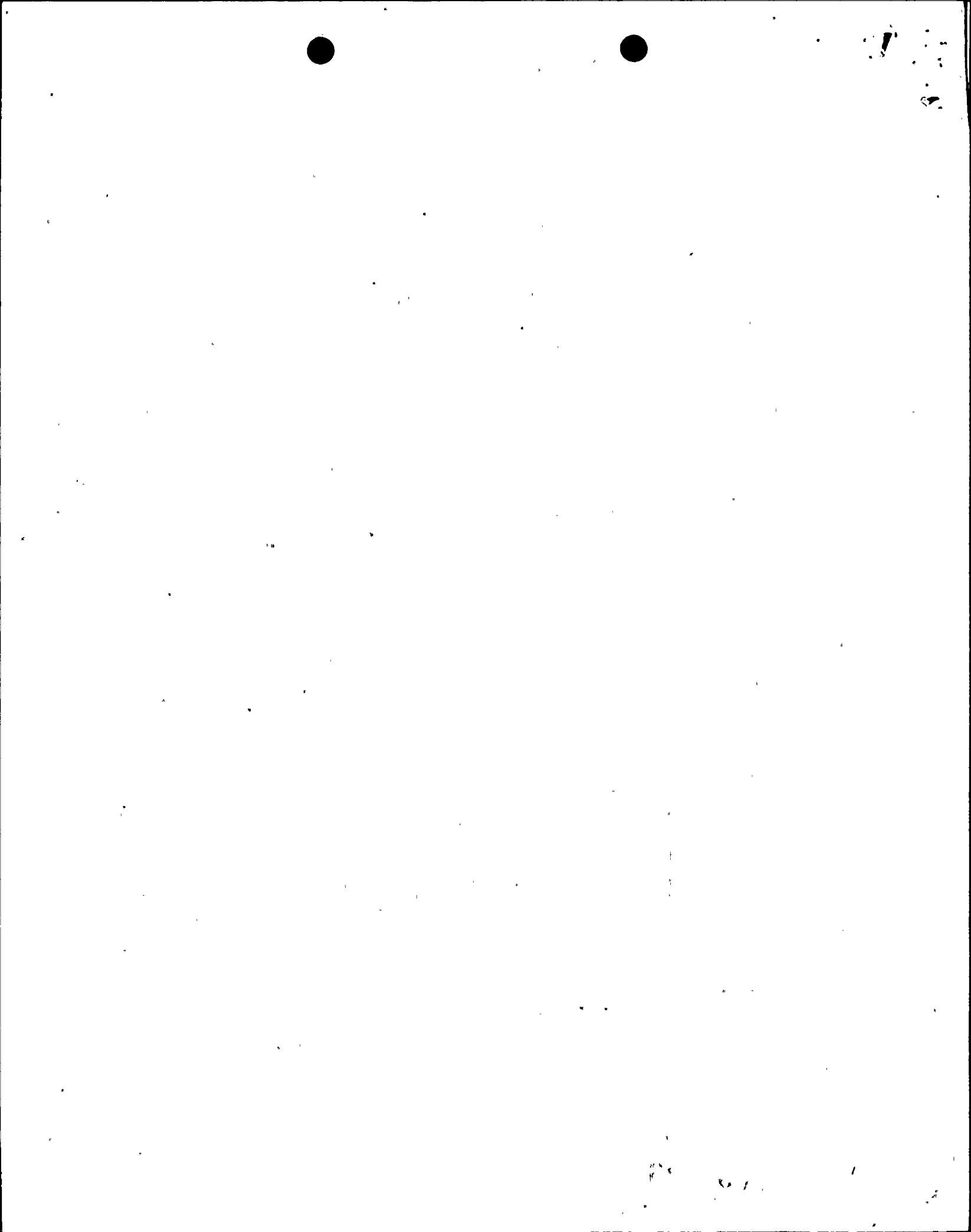
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6. Letter from J. A. Mitchell, NRC, to R. C. Mecredy, Rochester Gas and Electric Corporation, "R. E. Ginna - Acceptance for Referencing of Pressure Temperature Limits Report, Revision 1 (TAC No. M94770)," May 23, 1996.
7. Amendment No. 64 to Facility Operating License No. DPR-18, R. E. Ginna Nuclear Power Plant, May 23, 1996.
8. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation to G. S. Vissing, NRC, "Application for Amendment to Facility Operating License, Revision to Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR), Rochester Gas and Electric Corporation, R. E. Ginna Nuclear Power Plant, Docket No. 50-244," September 13, 1996. (WCAP-14684 Attached).
9. Letter from G. S. Vissing, NRC, to R. C. Mecredy, Rochester Gas and Electric Corporation, "R. E. Ginna - Acceptance of Request to Extend Time for Approval of Revision of Pressure and Temperature Limits Report (PTLR) (TAC No. M97313)," December 10, 1996.
10. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation to G. S. Vissing, NRC, "Application for Amendment to Facility Operating License, Revision to Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR) Administrative Controls Requirements, Rochester Gas and Electric Corporation, R. E. Ginna Nuclear Power Plant, Docket No. 50-244," April 24, 1997.
11. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation to G. S. Vissing, NRC, "Clarifications to Proposed Low Temperature Overpressure Protection (LTOP) Technical Specification, Rochester Gas and Electric Corporation, R. E. Ginna Nuclear Power Plant, Docket No. 50-244," June 3, 1997.
12. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation, "Request for Exemption from 10 CFR 50.60, Acceptance Criteria for Fracture Prevention for Light Water Nuclear Power Reactors for Normal Operation," June 12, 1997.

13. Letter from G. S. Vissing, NRC to R. C. Mecredy, Rochester Gas and Electric Corporation, "R. E. Ginna - Acceptance of Request to Extend Time for Approval of Revision of Pressure and Temperature Limits Report (PTLR) (TAC No. M98992)," June 26, 1997.
14. Letter from G. S. Vissing, NRC to R. C. Mecredy, Rochester Gas and Electric Corporation, "Exemption from the Requirements of 10 CFR Part 50.60, Acceptance Criteria for Fracture Prevention Measures for Lightwater Nuclear Power Reactors for Normal Operation - R. E. Ginna Nuclear Power Plant (TAC No. M98993)," July 28, 1997.
15. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation to G. S. Vissing, NRC, "Application for Amendment to Facility Operating License, Revision to Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR) Administrative Controls Requirements, Rochester Gas and Electric Corporation, R. E. Ginna Nuclear Power Plant, Docket No. 50-244," September 29, 1997.
16. Letter from R. C. Mecredy, Rochester Gas and Electric Corporation to G. S. Vissing, NRC, "Corrections to Proposed Low Temperature Overpressure Protection Technical Specifications, Rochester Gas and Electric Corporation, R. E. Ginna Nuclear Power Plant, Docket No. 50-244," October 8, 1997.

We have completed our review of the pressure temperature (P/T) and low overpressure protection (LTOP) system limits methodology and the pressure temperature limits report (PTLR) (as referenced above), submitted by the Rochester Gas and Electric Corporation. We find the methodology to be acceptable for referencing in the administrative controls section of the R. E. Ginna Nuclear Power Plant Technical Specifications (TS) to the extent specified and under the limitations delineated in your submittals and the associated NRC safety evaluation, which is enclosed. The safety evaluation defines the basis for acceptance of the submittals. Our acceptance only applies to the matters described in the submittals.

The methodology for review relating to the P/T and LTOP system limits was provided in the references listed above. Reference 3 included WCAP-14040, Revision 1 (WCAP-14040-NP-A), which provided parts of the methodology used for determining the acceptance of the R. E. Ginna methodology.

Robert C. Mecredy

- 4 -

The methodology in WCAP-14040, along with supplements provided by RG&E will be used to calculate future changes to the P/T and LTOP limits. RG&E may generate new P/T and LTOP system limits in accordance with this methodology without prior approval of the staff. System limits may be subject to audit by the staff through inspections as necessary.

We do not intend to repeat our review of the matters described in the submittals if the submittals appear as references in other license applications relating to your plant, except to ensure that the material in the submittals is still applicable to your plant as indicated in the conclusion section of the safety evaluation.

Should our criteria or regulations change so that our conclusions as to the acceptability of the methodology is invalidated, licensees referencing these documents will be expected to revise and resubmit their respective documentation, or submit justification for the continued effective applicability of the documents without revision of their respective documentation.

Sincerely,



S. Singh Bajwa, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: Safety Evaluation

cc w/encl: See next page



November 28, 1997

Robert C. Mecredy

- 4 -

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Sincerely,

Original Signed by:

S. Singh Bajwa, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: Safety Evaluation

cc w/encl: See next page

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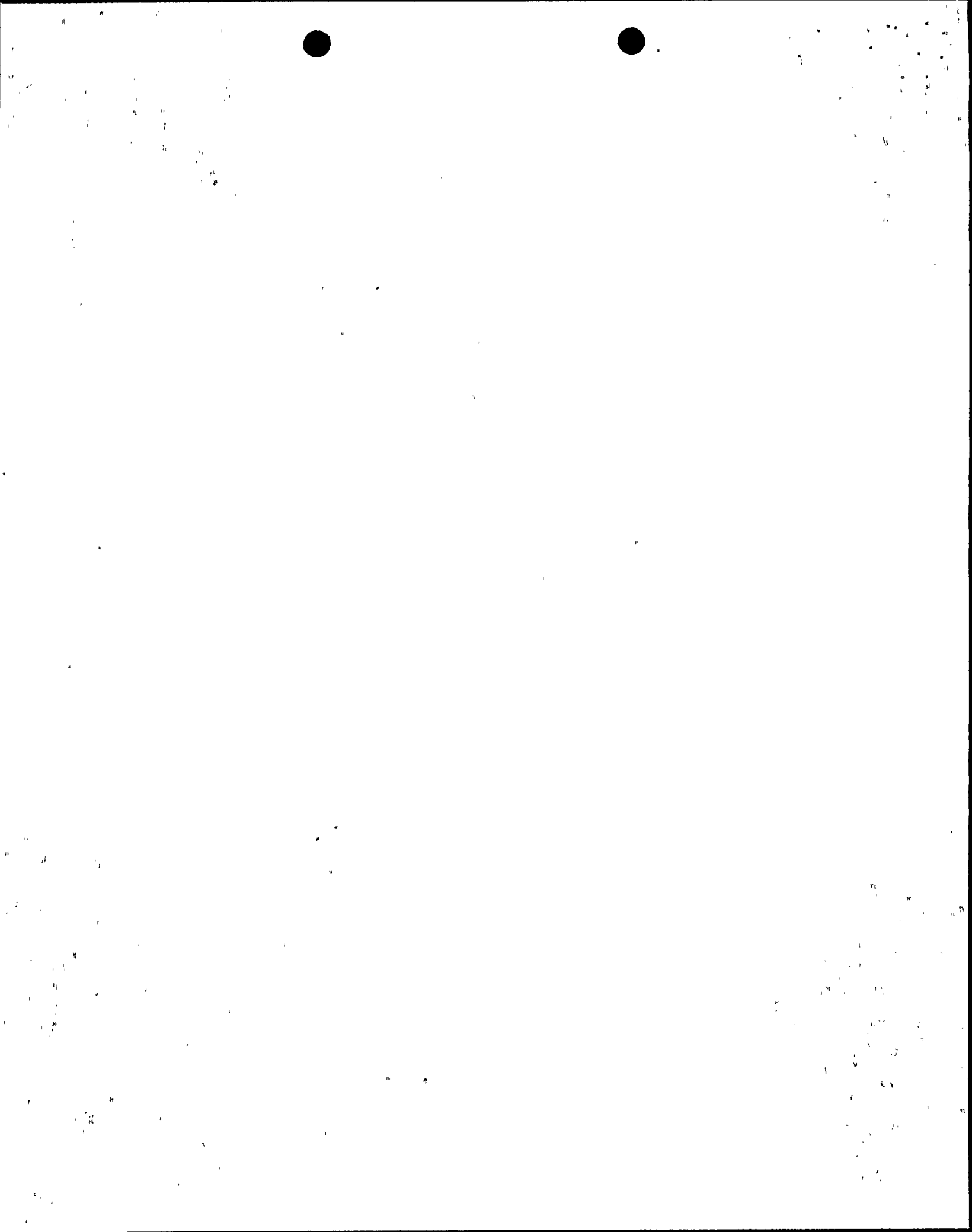
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Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

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