CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9707170124 DOC.DATE: 97/07/08 NOTARIZED: NO DOCKET #
FACIL:50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
AUTH.NAME AUTHOR AFFILIATION
MECREDY,R.C. Rochester Gas & Electric Corp.
RECIP.NAME RECIPIENT AFFILIATION
VISSING,G.

SUBJECT: Submits 30-day annual ECCS rept re error in LOCA analysis per 10CFR50.46,paragraph (a)(3)(ii).

DISTRIBUTION CODE: A001D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 5
TITLE: OR Submittal: General Distribution

NOTES:License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL
	PD1-1 LA	1	1	PD1-1 PD	1	1
	VISSING,G.	1	1			
INTERNAL:	FILE CENTER 01	1	1	NRR/DE/ECGB/A	1	1
	NRR/DE/EMCB	1	1	NRR/DRCH/HICB	1	1
	NRR/DSSA/SPLB	1	1	NRR/DSSA/SRXB	1	1
	NUDOCS-ABSTRACT	1	1	OGC/HDS3	1	0
EXTERNAL:	NOAC	1	1	NRC PDR	1	1

NOTE TO ALL "RIDS" RECIPIENTS:
PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM OWFN 5D-5(EXT. 415-2083) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 13 ENCL 12

CATEGORY 1 DOCUMENT

RGE AND

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001 AREA CODE 716 546-2700

ROBERT C. MECREDY
Vice President
Nuclear Operations

July 8, 1997

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: 10CFR50.46 30 Day and Annual ECCS Report
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref. (a): Westinghouse letter SAE-ESI-97-340, Subject: 10CFR50.46, Reporting Information, dated June 4, 1997.

(b) Westinghouse letter RGE-97-200, Subject 10CFR50.46 Annual Notification and Reporting, dated April 17, 1997.

Dear Mr. Vissing:

In accordance with the requirement in 10CFR50.46, paragraph (a)(3)(ii), this 30 day and annual ECCS report is hereby submitted.

Westinghouse, the provider of LOCA analysis for the R.E. Ginna Nuclear Power Plant, has provided RG&E with information regarding an error in the LOCA analysis that requires 30 day NRC notification (Reference a). In addition, References (a) and (b) provide an update to the peak cladding temperature (PCT) margin for 1996 and 1997 changes.

The large break LOCA PCT has increased by 58°F due to the misinterpretation/re-calculation of nominal initial accumulator water volume and transposition errors in transfer of code restart data. The new large break LOCA PCT is 2158°F and is summarized in Attachment 1 to this letter. The 58°F PCT penalty is the result of Ginna specific computer runs. The computer runs utilized the latest code version, a bounding initial accumulator water volume, implicitly corrects the restart data transfer errors, and corrects the errors reported in the 1995 Westinghouse Annual Report. Therefore, the current PCT is the result of direct plant specific sensitivity reanalysis. No schedule for reanalysis is proposed because Ginna specific analysis has been completed in determining the PCT penalty.

160055

The small break LOCA PCT has increased by 10°F due to an error in the fuel rod initialization process. The new small break LOCA PCT is 1323°F and is summarized in Attachment 1 to this letter.

Very truly yours,

Thomas A. Marlow for

Robert C. Mecredy

Attachment
RWE/465

xc: Mr. Guy Vissing (Mail Stop 14B2)
Project Directorate I-1
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Ginna Senior Resident Inspector

ATTACHMENT 1

LOCA PCT SUMMARY

JULY 1997 UPDATE

ATTACHMENT 1

LOCA PCT SUMMARY

Small Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: NOTRUMP Fuel: OFA
F_Q = 2.50 $F_{\Delta H}$ = 1.75 SGTP = 15%

A.	Analysis of Record (6/95) (effective 6/96)	PCT = 1308°F
B.	1995 10CFR50.46 Model Assessments	
	1. NOTRUMP Specific Enthalpy Error	ΔPCT = 20°F
	2. SALIBRARY Double Precision Errors	ΔPCT = -15°F
C.	1996 10CFR50.46 Model Assessments	
	1. SBLOCA Fuel Rod Initialization Error	ΔPCT = +10°F
D.	1997 10CFR50.46 Model Assessment	
	1. none	ΔPCT = 0°F
E.	Other margin allocations	
	1. none	ΔPCT = 0°F
	Licensing Basis	PCT = 1323°F

Revision Date: 7/97

ATTACHMENT 1

LOCA PCT SUMMARY

Large Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: UPI SECY Fuel: OFA
F_Q = 2.45 $F_{\Delta H}$ = 1.75 SGTP = 15%

A.	Analysis of Record (5/95) (effective 6/96)	PCT = 2051°F
B.	1995 10CFR50.46 Model Assessments	
	1. Fixed heat transfer node assignment error/Accumulator water injection error	ΔPCT = 48°F
C.	1996 10CFR50.46 Model Assessments	
	1. none	ΔPCT = 0°F
D.	1997 10CFR50.46 Model Assessments	
	1. Accumulator Initial Water Volume Restart Data Transposition Error Plant Specific Analytical Reassessment of 1995 Model Assessments	ΔPCT = 58°F
E.	10CFR50.59 Evaluations	
	1. Service Water Temp. ≥ 30°F (1997 evaluation; SEV-1090)	ΔPCT = 1°F
	Licensing Basis	PCT = 2158°F

Revision Date: 7/97