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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001 November 6, 1996

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50-24

LICENSEE: Rochester Gas and Electric Corporation

- FACILITY: R. E. Ginna Nuclear Power Plant
- SUBJECT: SUMMARY OF MEETING WITH REPRESENTATIVES OF ROCHESTER GAS AND ELECTRIC CORPORATION JULY 10, 1996, CONCERNING A PROPOSED RERACKING OF THE SPENT FUEL STORAGE POOL WITH BORATED STAINLESS MATERIALS (TAC NO. M95759)

In preparation for the planned reracking of the Ginna Spent Fuel Storage Pool (SFSP), representatives of Rochester Gas and Electric Corporation (RG&E) together with representatives of RG&E vendors, Framatome Technologies, Inc. (FTI), Framatome Cogema Fuels (FCF) and Societe Atlantique de Techniques Avancees (ATEA) presented plans for expanding the storage capacity of the Ginna SFSP. The meeting discussed proprietary issues. This summary addresses the non-proprietary issues. Enclosure 1 provides the agenda of the meeting, Enclosure 2 provides the list of attendees of the meeting, and Enclosure 3 provides the viewgraphs of the presentation materials. Enclosure 4 provides the proposed schedule for the total project.

The current configuration of the Ginna SFSP consists of two regions. Region 1 consists of stainless steel racks with 176 storage locations in a checker board pattern. Region 2 consists of stainless steel racks with boraflex and with 840 storage locations. It is proposed to replace the Region 1 racks with borated stainless steel racks. Two locations are proposed in Region 1. One with borated stainless steel that would accommodate 187 storage locations and one with borated stainless steel in a checker board pattern that would accommodate 292 storage locations. These would be installed during this reracking. There would be additional borated stainless steel racks that would be placed along sides of the present Region 2 racks. The construction of these racks would be at a later date.

Several concerns were discussed including (1) potential loading conditions for future spent fuel casks designs as they affect the dropping of heavy loads analyses, (2) adequacy of the number of feet per rack, (3) provisions for preventing tipping of a rack during a seismic event, (4) concern for thermal expansion that could cause buckling of the borated stainless steel plates in the lateral direction, and (5) concern for the corrosion resistance of the material through all temperature ranges that the material would experience. Of prime concern was adequacy of the stiffness of the racks. The staff would like to see a demonstration test that would verify the adequacy of the stiffness of the structures.

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November 6, 1996

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The submittal for a license amendment is planned for early January 1997 with completion of NRC review by early November 1997. This would accommodate fabrication, construction at the site and completion by late December 1998 which would accommodate the refueling of the reactor in March 1999.

/S/

Guy S. Vissing, Senior Project Manager Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-244

- Enclosures: 1. Agenda
 - 2. List of Attendees
 - 3. Viewgraphs of Presentation Materials
 - 4. Project Schedule

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