Docket File,



UNITED STATES NUCLEAR REGULATORY COMMISSION

۱.

WASHINGTON, D.C. 20555-0001

June 21, 1995

Dr. Robert C. Mecredy Vice President, Nuclear Operations Rochester Gas and Electric Corporation 89 East Avenue Rochester, NY 14649

SUBJECT: R. E. GINNA NUCLEAR POWER PLANT INSERVICE TESTING (IST) PROGRAM FOR PUMPS AND VALVES (1990-1999 THIRD 10-YEAR INTERVAL) RELIEF REQUEST NOS. VR-31 AND VR-32 (TAC NO. M89336)

Dear Dr. Mecredy:

9507120342

ADOCK

The NRC staff has reviewed information provided by you in your letter dated April 15, 1994, related to the above subject relief requests for the R. E. Ginna Nuclear Power Plant IST Program for pumps and valves.

Revision 2 of the R. E. Ginna Nuclear Power Plant IST Program for pumps and valves was submitted in your letter of April 15, 1994. Revision 2 incorporates all of the actions taken to address IST Program anomalies identified in the NRC's safety evaluations (SEs) dated April 15, 1991, and October 20, 1992, with NRC review completed in an SE issued November 4, 1993. Additionally, Revision 2 included two new relief requests involving leakage testing of containment isolation valves added to the IST Program, which are now evaluated. The IST program was developed to the 1986 Edition of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code).

Relief Request No. VR-31 in essence is a request to use a later portion of the ASME Code and is granted pursuant to the provisions of 10 CFR 50.55a(f)(4)(iv) provided the requirements in paragraph 4.2.2.3 of Part 10 are included in the implementation of the alternative as discussed in the enclosed SE. The rules in OM-10 for leak testing valve combinations may not be consistent with the requirements in 10 CFR Part 50, Appendix J; therefore, you should determine if the requirements of Appendix J are met, and if not, take appropriate actions. The approval pursuant to 10 CFR 50.55a(f)(4)(iv) does not constitute approval of an exemption to Appendix J.

Relief Request No. VR-32 and its alternative is granted pursuant to 10 CFR 50.55a(f)(6)(i) based on the impracticality of the design of the piping configuration and the burden of modifying the design if the ASME Code requirements were imposed, provided you perform an evaluation to determine if the measured leakage rate or the acceptance criteria must be adjusted because of the remaining water in the piping. The evaluation of this relief request, however, is not applicable to the testing requirements for local leak rate testing of this valve pursuant to 10 CFR Part 50, Appendix J. If you

NRC FILE CENTER COPY

R. Mecredy

3

determine that an exemption to Appendix J is needed and has not been requested, actions should be taken as appropriate.

The enclosed SE provides the results of the review and TAC No. M89336 is considered complete.

Sincerely,

march

Ledyard B. Marsh, Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: Safety Evaluation

cc w/encl: See next page

Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

cc:

Thomas A. Moslak, Senior Resident Inspector R.E. Ginna Plant U.S. Nuclear Regulatory Commission 1503 Lake Road Ontario, NY 14519

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

Mr. William Valentino, President New York State Energy, Research, and Development Authority 2 Rockefeller Plaza Albany, NY 12223

Charlie Donaldson, Esq. Assistant Attorney General New York Department of Law 120 Broadway New York, NY 10271

Nicholas S. Reynolds Winston & Strawn 1400 L St. N.W. Washington, DC 20005-3502

Ms. Thelma Wideman Director, Wayne County Emergency Management Office Wayne County Emergency Operations Center 7336 Route 31 Lyons, NY 14489

Ms. Mary Louise Meisenzahl Administrator, Monroe County Office of Emergency Preparedness 111 West Fall Road, Room 11 Rochester, NY 14620