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ROBERTIC MECREDY Vice President Nuclear Operations

July 7, 1995

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U.S. Nuclear Regulatory Commission Document Control Desk Attn: Allen R. Johnson PWR Project Directorate I-1 Washington, D.C. 20555

Subject: LER 95-005, Instrument Air Leak in Containment Causes Feedwater Isolation R.E. Ginna Nuclear Power Plant Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (iv), which requires a report of, "Any event or condition that resulted in a manual or automatic actuation of any engineered safety feature (ESF), including the reactor protection system (RPS)", the attached Licensee Event Report LER 95-005 is hereby submitted.

This event has in no way affected the public's health and safety.

Very truly yours,

Robert C. Mecredy

xc: U.S. Nuclear Regulatory Commission
 Mr. Allen R. Johnson (Mail Stop 14B2)
 PWR Project Directorate I-1
 Washington, D.C. 20555

U.S. Nuclear Regulatory Commission Region I 475 Allendale Road King of Prussia, PA 19406

Ginna USNRC Senior Resident Inspector

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NRC FORM	366			U.S.	NUCLEAR	REGUL	ATORY	CONH	ISSION			APPROVED BY EXPI	OMB NO. RES 5/31/	3150-0 95	104	
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TITLE (4	;) ;)	Ins	trume	ent Air Lea	ık in (	Cont	tain	mer	it Ca	ause	es	Feedwate	er Isc	lati	ion	
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NRC FORM	566A		U.S. NUCLEAR RE	GULATORY COMMISSION		APPROVED BY C EXPIRE	MB NO. 315 S 5/31/95	0-0104		
(3-92)		LICE	N <mark>SEE EVENT REPORT (LE</mark> F TEXT CONTINUATION	2)	ESTIMAT THIS IN FORWARD THE IN (MNBB 7 WASHING REDUCTI MANAGEM	ED BURDEN PER NFORMATION COLLE COMMENTS REGA FORMATION AND F 714), U.S. NUCLI TON, DC 20555-0 ON PROJECT ENT AND BUDGET,	RESPONSE CTION REQ RDING BURE RECORDS MA EAR REGULAT 001, AND T (3150-0104) WASHINGTON	TO CO UEST: IEN ES NAGEMEN IORY CC O THE ), OI N. DC 2	MPLY 50.0 TIMATI NT BR MMISS PAPER FFICE 20503.	WITH HRS. E TO KANCH SION, WORK OF
		FAC	ILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	)	PA	GE (3	5)
R.E.	Gir	nna 1	Nuclear Power Plant	05000244	year 95	SEQUENTIAL NUMBER 005	REVISION NUMBER 00	2	OF	8
TEXT (If n	pore sp	ace is	required, use additional copies of	NRC Form 366A) (17	()					
I.	PRE	-EVE	NT PLANT CONDITIONS:	1	)					
	The sig the in com	pla nifi Ins Inst pone	nt was at approximate cant activities in p strument Air system i rument Air pressure, ents, including the ma	rogress. A n Containmen and loss of ain feedwater	sold sold nt fa f con c regu	ate reacto ered join iled, cau trol air ulating va	t conn sing a to air alves.	ecti dec :-ope	ion crea erat	no in ase ced
II.	DES	CRIF	TION OF EVENT:							
	А.	DAT	ES AND APPROXIMATE TI	MES OF MAJOR	occu	IRRENCES:			,	
		0	June 7, 1995, 1856 E Containment.	DST: Instrum	nent <i>l</i>	Air system	a leak	οςςι	ırs	in
		0	June 7, 1995, 1902 isolated, restoring components outside C	EDST: Inst normal ai ontainment.	rumen .r pi	t Air to ressure t	Conta coair	inme :-ope	nt erat	is ced
		0	June 7, 1995, 190 increase above 67%.	5 EDST: St Event date	eam: and t	Generator time.	s (s/g	;) ]	leve	els
		0	June 7, 1995, 1905 E	DST: Discove	ry da	ate and ti	me.			
		0	June 7, 1995, 1910 pre-event normal ope	EDST: "A" and rating band.	nd "B	" S/G lev	els re	stor	ced	to
	в.	EVE	NT:							
		On app con (CN dec in beg the (MF IA	June 7, 1995, at appr roximately 97% steady nection on a two inch MT) failed, resulting rease in IA pressure. loss of control air t inning to travel to t se components were th RV) (fail closed) whi pressure at the valve	oximately 18 state react Instrument in leakage This decre o air-operat heir respect e two main f ch drifted t actuator de	56 ED or pc Air ( from ase i ed cc ive " eedwa owarc creas	ST, with wer, a so [IA] line the IA sy in IA pres omponents, fail" pos ter regul s the clo sed.	the pl ldered in Con stem a sure r with itions ating sed po	ant joi tain nd esul valv valv siti	at nt men ted won es on	g as

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NRC FORM 366A U.S. NUCLEAR	REGULATORY CONHISSION	APPROVED BY O EXPIRE	MB NO. 3150-0104 S 5/31/95
LICENSEE EVENT REPORT (LI TEXT CONTINUATION	SR)	ESTIMATED BURDEN PER THIS INFORMATION COLLE FORWARD COMMENTS REGA THE INFORMATION AND F (MNBB 7714), U.S. NUCLE WASHINGTON, DC 20555-0 REDUCTION PROJECT MANAGEMENT AND BUDGET,	RESPONSE TO COMPLY WITH CTION REQUEST: 50.0 HRS. RDING BURDEN ESTIMATE TO ECORDS MANAGEMENT BRANCH AR REGULATORY COMMISSION, 001, AND TO THE PAPERWORK (3150-0104), OFFICE OF WASHINGTON, DC 20503.
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)
R.E. Ginna Nuclear Power Plant	05000244	YEAR SEQUENTIAL NUMBER 95 005	NUMBER 00 3 OF 8
TEXT (If more space is required, use additional copies	of NRC Form 366A) (17	7)	
At approximately 1902 diagnosed the probable CNMT Isolation valve, the leak was isolated, components outside CNMT	EDST, the location of t AOV-5392. W and normal	Control Room the IA leak, an ith the closur IA pressure w	operators had d closed the IA re of AOV-5392, vas restored to
Due to the MFRVs drift. Generator (S/G) levels "demand" signal to the in restoration of IA responding to the incre went to the full open the "A" S/G and 40% i resulted in increasing	ing closed, f s decreased, MFRVs. Isol pressure, a eased demand s position, le n the "B" S/ level in the	eedwater (FW) resulting in ation of the I and the MFRVs signal. At the evel was approx 'G. The incre "A" and "B" S/	flows and Steam an increasing A leak resulted opened fully, time the MFRVs ximately 25% in ease in FW flow Gs.
Within three minutes increased to cause FW I level >/= 67 % narrow response to this FW I when level decreased b there were several occu level cycled around 67 the "A" S/G also increa the "A" S/G. For a occurrences of FW Isc around 67%. This shor the Control Room oper levels. At approximat S/Gs were restored to t	narrow range solation on h v range level solation sign below 67%. urrences of FV %. During th ased to cause approximately blation for t t term S/G le ators took m cely 1910 EDS cheir normal c	e level in the high level in the high level in the l). The "B" hal as designed For the next N Isolation for is time narrow FW Isolation of twenty secon the "A" S/G a evel transient anual control T, levels in the perating band.	e "B" S/G had the "B" S/G (S/G MFRV closed in d, and reopened ninety seconds, the "B" S/G as range level in on high level in ds, there were is level cycled continued until to restore S/G the "A" and "B"
C. INOPERABLE STRUCTURES, THE EVENT:	COMPONENTS, (	OR SYSTEMS THAT	CONTRIBUTED TO
The decrease in IA pro air-operated components	essure result 3.	ed in loss of	control air to
D. OTHER SYSTEMS OR SECONI	DARY FUNCTIONS	5 AFFECTED:	
Due to the failed so subsequent isolation of CNMT failed to their re- several valves and ven Compartment Cooling (R dampers failed closed.	ldered joint of IA to CNM espective "fa tilation damp CC) fan moto:	connection in F, air-operate il" positions. ers. In addit: r tripped when	CNMT, and the d components in These included ion, the Reactor the associated

NRC FORM 366A (5-92)	U.S. NUCLEAR RE	GULATORY CONNISSION	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95				
	LICENSEE EVENT REPORT (LEF TEXT CONTINUATION	8)	ESTIMAT THIS I FORWARD THE IN (MNBB 7 WASHING REDUCTI MANAGEM	TED BURDEN PER NFORMATION COLLE COMMENTS REGA FFORMATION AND F 7714), U.S. NUCLI 310N, DC 20555-0 10N PROJECT 4ENT AND BUDGET,	RESPONSE ECTION REQU RDING BURD RECORDS MAI EAR REGULAT 001, AND T (3150-0104) WASHINGTON	TO COMPLY UEST: 50.0 DEN ESTIMAT NAGEMENT B TORY COMMIS O THE PAPE D, OFFICE I, DC 20503	WITH HRS. TE TO RANCH SION, RWORK OF
	FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6	)	PAGE (	3)
R.E. G	inna Nuclear Power Plant	05000244	year 95	SEQUENTIAL NUMBER	REVISION NUMBER 00	4 OF	8
TEXT (16 mana	annea in populard, use additional copies of	NPC Form 3664) (17	<u> </u>	005			

For air-operated components outside the CNMT, there was a decrease in IA pressure throughout the plant for six minutes, until IA to CNMT was isolated. During this time, numerous airoperated components outside CNMT started to travel to their respective "fail" positions. With the exception of the MFRVs, this loss of control air did not adversely affect the ability of the Control Room operators to maintain plant conditions.

E. METHOD OF DISCOVERY:

This event was immediately apparent due to alarms and indications in the Control Room. In particular, Main Control Board annunciators C-17 (CONTAINMENT VENT SYSTEM) and H-8 (INSTRUMENT AIR LO PRESS 100 PSI) alarmed, indicating a problem with IA in CNMT.

F. OPERATOR ACTION:

The Control Room operators responded to Main Control Board annunciators C-17, H-8 and H-16 (INSTRUMENT AIR COMP), and referred to Alarm Response Procedures C-17, H-8 and H-16. They entered Abnormal Operating Procedure AP-IA.1 (LOSS OF INSTRUMENT AIR). The Control Room operators requested that the auxiliary operator start the standby diesel-driven air compressor. Following the steps of AP-IA.1 and with the knowledge that abnormal alarms were received on CNMT systems prior to those on secondary systems, the Control Room operators isolated IA to CNMT by closing the IA CNMT Isolation valve (AOV-5392). This action isolated the leak from the rest of the IA system, and IA pressure increased to normal pressure in the rest of the system.

After the FW Isolation, the Control Room operators transferred control of the MFRVs to "manual" to restore S/G levels to their normal operating band. When S/G levels and FW flows were stabilized, they transferred control of the MFRVs back to "automatic".

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NRC FORM 366A	U.S. NUCLEAR RE	GULATORY COMMISSION		APPROVED BY O EXPIRE	MB NO. 315 S 5/31/95	0-0104	
	LICENSEE EVENT REPORT (LEF TEXT CONTINUATION	ι)	ESTIMATED BURDEN PER RESPONSE TO CO THIS INFORMATION COLLECTION REQUEST: FORWARD COMMENTS REGARDING BURDEN ES THE INFORMATION AND RECORDS MANAGEME (MNBB 7714), U.S. NUCLEAR REGULATORY CO WASHINGTON, DC 20555-0001, AND TO THE REDUCTION PROJECT (3150-0104), OI MANAGEMENT AND BUDGET, WASHINGTON, DC 20 100000000000000000000000000000000000				WITH HRS. TO ANCH ION, WORK OF
	FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	)	PAGE (3)	<u>&gt;</u>
R.E. Gir	nna Nuclear Power Plant	05000244	YEARSEQUENTIAL NUMBERREVISION NUMBER9500500			5 OF	8
EXT (If more spa	ace is required, use additional copies of	NRC Form 366A) (17)	>				
	With loss of letdown charging flow to minimu The Shift Supervisor reduction until the lead returned to normal throu EDST, a power reduction per Normal Operating Pro Subsequently, the Contr personnel and higher sup An auxiliary operator conducted a CNMT entry isolate the leak. The header in CNMT. A tempo connection. This repai restore some pressure t allow operation of selec The NRC Operations Cente EDST, per 10CFR50.72 (b) notification.	flow, the m flow, and made a dec k was locate ighout the s was started cedure O-5.1 col Room op ervision. and Radia at power in leak was loc brary repair r enabled t co the IA s ted valves a r was notifi (2) (ii), n	operation d sector sision ed, is ystem d at (LOA perato ation he co ystem ind ve ed at	tors manu ured one to ini solated, . At app one perce D REDUCTI rs notif: Protect attempt for on the ma made to t on the ma made to t ontrol Roo in CNMT, ntilation approxim ergency f	ally charg and IA oroxima ent pe ONS). ied ma ion t to ide ain tw he fai om ope suff dampe ately our ho	decreas ing pum a pow pressu tely 19 r minut intenan cechnici ntify a o inch led joi rators icient rs. 2211 ur	ed pereso e, and Into
G.	SAFETY SYSTEM RESPONSES:						
	The MFRVs and MFRV bypas of the FW Isolation sign Containment Recirculatio their respective safegua charging and letdown als	s valves clo als. Ventil n Cooling fa rds position o failed to	sed a ation ns an s. C their	utomatica dampers d RCC fan NMT Isola safeguar	lly as for th s fail tion v ds pos	a resu e ed to alves fo itions.	lt or
III. CAU	JSE OF EVENT:						
A.	IMMEDIATE CAUSE:						
	The immediate cause of t being >/= 67%. The high when the MFRVs went full signal. This situation	he FW Isolat level was c open in res resulted in	ion w aused ponse overf	as level by incre to the v eeding th	in the ased F alve d e S/Gs	S/Gs W flows emand •	

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NRC FORM 366A (5-92)

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NRC FO (5-92)	rň 366a	U.S. NUCL	LEAR REC	SULATORY COMMISSION	APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95					
	LICEN	SEE EVENT REPORT TEXT CONTINUATIO	(ler N	)	ESTIMAT THIS I FORWARD THE IN (MNBB 7 WASHING REDUCTI MANAGEM	TED BURDEN PER NFORMATION COLLE COMMENTS REGA FORMATION AND F 7714), U.S. NUCLI STON, DC 20555-0 ION PROJECT TENT AND BUDGET,	RESPONSE ECTION REOU RDING BURD RECORDS MAI EAR REGULAT 001, AND T (3150-0104) WASHINGTON	TO COMPL JEST: 50. EN ESTIM NAGEMENT ORY COMM O THE PAR O, OFFIC I, DC 2050	Y WITH O HRS. ATE TO BRANCH ISSION, PERWORK CE OF D3.	
	FACI	LITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6		PAGE	(3)	
R	E. Ginna Nu	uclear Power Plar	at	05000044	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
				05000244	95	005	00		F. Q	

**B. INTERMEDIATE CAUSE:** 

The intermediate cause of the open valve demand signal for the MFRVs was decreased FW flows and S/G levels as the MFRVs drifted toward the closed position as IA pressure decreased.

C. ROOT CAUSE:

The underlying cause of the decrease in IA pressure was the failure of a soldered joint connection in a two inch IA line in CNMT. This was caused by insufficient insertion of the pipe into a fitting during original construction. This event is NUREG-1022 (B), "Design, Manufacturing, Code Construction Cause Installation". This event does not meet the NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Nuclear Power Plants", definition · Maintenance at of а "Maintenance Preventable Functional Failure".

IV. ANALYSIS OF EVENT:

This event is reportable in accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (iv), which requires a report of, "Any event or condition that resulted in a manual or automatic actuation of any engineered safety feature (ESF), including the reactor protection system (RPS)". The FW Isolation of the "A" and "B" S/Gs was an automatic actuation of an ESF system.

An assessment was performed considering both the safety consequences and implications of this event with the following results and conclusions:

There were no operational or safety consequences or implications attributed to the FW isolations because:

- The FW isolations occurred at the required S/G level.
- S/G levels were quickly stabilized and manual control of MFRVs was accomplished to mitigate any consequences of the event.

Based on the above, it can be concluded that the public's health and safety was assured at all times.

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ARC FORM 366A	U.S. NUCLEAR R	EGULATORY COMMISSION	APPROVED BY ONB NO. 31 EXPIRES 5/31/95	50-0104
	ICENSEE EVENT REPORT (LE) TEXT CONTINUATION	R)	ESTIMATED BURDEN PER RESPONSE THIS INFORMATION COLLECTION REG FORWARD COMMENTS REGARDING BUR THE INFORMATION AND RECORDS M (MNBB 7714), U.S. NUCLEAR REGULA WASHINGTON, DC 20555-0001, AND REDUCTION PROJECT (3150-0104 MANAGEMENT AND BUDGET, WASHINGTO	TO COMPLY WITH DUEST: 50.0 HRS. DEN ESTIMATE TO ANAGEMENT BRANCH TORY COMMISSION, TO THE PAPERWORK (), OFFICE OF N. DC 20503.
<u> </u>	FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)
R.E. Gin	na Nuclear Power Plant	05000244	YEAR SEQUENTIAL REVISION NUMBER NUMBER 95 005 00	- 7 OF 8
TEXT (If more spa	nce is required, use additional copies o	f NRC Form 366A) (17	7)	
V. CORI	RECTIVE ACTION:			, ,
А.	ACTION TAKEN TO RETURI STATUS:	N AFFECTED	SYSTEMS TO PRE-EVE	NT NORMAL
	o MFRVs were returne restored to their pr	ed to autom re-event norm	atic after S/G le al operating band.	vels were
	<ul> <li>A temporary repair w CNMT. This temporators to restore sufficient to all ventilation dampers.</li> </ul>	was made to t prary repair e some press ow operatio	the failed joint con enabled the Con ure to the IA system on of selected va	nection in trol Room n in CNMT, alves and
	<ul> <li>After letdown flow operators could control</li> <li>reduction was stopped</li> </ul>	w was rest ontrol prima ed.	ored and the Con ry system volume,	trol Room the load
	Maintenance personn designed by Engine failed joint for pe supply to the letdo maintain letdown flo	nel installe ering, which ermanent repa wn valves. ow.	ed a temporary mo permitted isolation air, while maintaini This allowed the op	dification on of the ng an air erators to
	Maintenance person replacing the fail sections, and remove completion of these CNMT.	nel performe Led joint c ved the temp e activities	ed the permanent onnection and adja oorary modification. , normal IA was re	repair by cent pipe At the estored to
в.	ACTION TAKEN OR PLANNED	TO PREVENT R	ECURRENCE:	
	O A sample of joint examined by non-dea pipe insertion into	connections structive te fittings.	s in the IA system chniques to confirm	n will be n adequate
	• This event will b Simulator response learned and enhance pressure will be appropriate.	e evaluated under contro ements to th identified,	and compared agai lled conditions. A ne control of prima and procedures ch	nst Plant ny lessons ary system anged, as

NRC FORM 366A (5-92)	U.S. NUCLEAR R	EGULATORY COMMISSION	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95				
	LICENSEE EVENT REPORT (LE) TEXT CONTINUATION	R)	ESTIMAT THIS I FORWARD THE IN (MNBB WASHIND REDUCT MANAGEN	TED BURDEN PER NFORMATION COLLE COMMENTS REGAN FORMATION AND F 7714), U.S. NUCLI STON, DC 20555-0 ION PROJECT ( HENT AND BUDGET,	RESPONSE CTION REQU RDING BURD RECORDS MAI EAR REGULAT 001, AND T (3150-0104) WASHINGTON	TO COMPLY WITH JEST: 50.0 HRS. EN ESTIMATE TO NAGEMENT BRANCH ORY COMMISSION, O THE PAPERWORK OFICE OF , DC 20503.	
	FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	)	PAGE (3)	
R.E. Gi	nna Nuclear Power Plant	05000244	year 95	8 OF 8			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

## VI. ADDITIONAL INFORMATION:

A. FAILED COMPONENTS:

There were no component failures, in that the leak occurred when a soldered joint connection failed. This joint connected a two inch copper pipe to a two inch copper elbow fitting. The manufacturer of the pipe and fitting is not relevant, and the manufacturer of the solder is unknown.

B. PREVIOUS LERS ON SIMILAR EVENTS:

A similar LER event historical search was conducted with the following results: No documentation of similar LER events with the same root cause at Ginna Nuclear Power Plant could be identified.

C. SPECIAL COMMENTS:

None