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 MODES, M.C. Region 1 (Post 820201)
 RECIP. NAME RECIPIENT AFFILIATION
 MECREDDY, R.C. Rochester Gas & Electric Corp.

SUBJECT: Forwards insp rept 50-244/95-11 on 950501-03. Non-cited violation noted. Util evaluation of root cause leading to defective sleeve welds in SG acceptable.

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June 5, 1995

DEI

Dr. Robert C. Mecredy
Vice President, Ginna Nuclear Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

SUBJECT: NRC REGION I INSPECTION REPORT 50-244/95-11

Dear Dr. Mecredy:

This letter transmits the findings of the announced safety inspection conducted by Mr. Prakash Patnaik of this office from May 1-3, 1995, at Ginna Nuclear Power Station, Ontario, New York.

The inspection was directed at areas important to public health and safety, through reviews of the inservice inspection program, and the steam generator eddy current examination program during the spring 1995 outage of the unit. The inspector also assessed your evaluation of deficient conditions in some of the steam generator upper sleeve welds found during eddy current examination of sleeved steam generator tubes.

The inservice inspection program and the eddy current examination program for Ginna were found to be comprehensive and in some instances exceeded the requirements of the applicable ASME code, Section XI and the technical specifications. Your evaluation of the root cause leading to defective sleeve welds in the steam generator is considered to be acceptable, based on inspector's review of the pertinent records and interviews with personnel.

The incomplete tube sleeve weld and subsequent failure of the ultrasonic examination violate a number of criteria, including Ginna UFSAR paragraph 5.4.2.7.2, (Rev. 10, 12/93) and 10 CFR Part 50, Appendix B Criteria IX, "Control of Special Processes." Because Rochester Gas and Electric identified the incomplete weld, it could not be reasonably expected to have been prevented by Ginna corrective action for a previous violation (because there weren't any); it was corrected in a reasonable time; and it was not willful; (10 CFR Part 2, Appendix C, B(2)) this is a Noncited Level IV Violation.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

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Dr. Robert C. Mecredy

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June 5, 1995

No response to this letter is required. Your cooperation with us is appreciated.

Sincerely,

Michael C. Modes, Chief
Materials Section
Division of Reactor Safety

Docket No. 50-244

Enclosure: Region I Inspection Report 50-244/95-11
w/Attachment

cc w/encl:

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Director, Energy and Water Division
State of New York, Department of Law
N. Reynolds, Esquire
D. Screnci, PAO (2)
NRC Resident Inspector
State of New York, SLO Designee

Dr. Robert C. Mecredy

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June 5, 1995

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