



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO APPENDIX J, EXEMPTION REQUEST AND

AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated March 13, 1995, the Rochester Gas and Electric Corporation (RG&E or the licensee) submitted a request for changes to the R. E. Ginna Nuclear Power Plant (Ginna), Technical Specifications (TSs). The requested changes would revise TS 4.4.2.4. to replace specific leakage frequencies for containment isolation with a reference to 10 CFR Part 50, Appendix J, Section III.D.3. RG&E requested these changes to the TS to support a one-time exemption from two parts of 10 CFR Part 50, Appendix J, Section III.D.3. These changes are administrative in nature.

The proposed language for TS 4.4.2.4.a is:

... individual penetrations and containment isolation valves shall be tested in accordance with 10 CFR Part 50, Appendix J, as modified by approved exemptions.

TS 4.4.2.4.a currently states:

... individual penetrations and containment isolation valves shall be tested during each reactor shutdown for refueling, or other convenient intervals, but in no case at intervals greater than 2 years.

The exemption to Appendix J, Section III.D.3 relieves RG&E from performing Type C tests during the 1995 refueling outage except for isolation valves which have maintenance performed on them or valves which have not demonstrated acceptable leakage during the previous two leakage rate tests. Second, the exemption relieves RG&E from the requirement to perform Type C tests within a 2-year interval. RG&E has been granted up to a 1-month extension of the 2-year interval for 129 containment isolation valves. Appendix J requires Type C testing for the 1995 refueling outage to be completed prior to exiting cold shutdown which is tentatively scheduled for April 27, 1995. RG&E will continue to perform Type B testing as required by Appendix J and approved exemptions, as well as the proposed TS, which reference Appendix J requirements.

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In addition, the current TS permits Type C tests at "other convenient intervals." This contradicts Appendix J which requires Type C testing during reactor shutdowns for refueling. Thus, the TS contradicts the regulation and is not valid. RG&E's proposed change to the TSs fixes this problem.

2.0 EVALUATION

The TS do nothing more than reference the regulations pertaining to Type C testing. The TS do not need to duplicate specific terms in the regulations. The TS also encompass the recent Appendix J exemption referenced above.

The NRC staff finds that RG&E's proposal is acceptable for several reasons. As discussed in RG&E's March 15, 1995, letter, the performance of the containment isolation valves and the Ginna overall containment integrity have been good. The as-left Type A test leakage rate is 35% of L. The current Type B and C as-left maximum path leakage rate is 61% of the 0.6 L Appendix J limit. Furthermore, RG&E has proposed to limit the exemption only to those valves on which no maintenance has been done and which have passed the last two consecutive Type C leakage rate tests. The staff has granted similar requests in the past. On February 2, 1994, the staff granted a similar exemption to the River Bend Station licensee, and by a letter dated April 29, 1987, the staff granted a similar request to the Washington Nuclear Power Plant, Unit 2 licensee.

As noted above, the present TS 4.4.2.4.a is inconsistent with Appendix J. RG&E has proposed to revise TS 4.4.2.4.a to make the wording consistent with Appendix J. the proposed TS is:

... individual penetrations and containment isolation valves shall be tested in accordance with 10 CFR Part 50, Appendix J, as modified by approved exemptions.

For the reasons set forth above, the NRC staff finds this TS change to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the

amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 15167). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Lobel

Date: April 26, 1995

April 26, 1995

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Allen R. Johnson, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 59 to License No. DPR-18
2. Safety Evaluation

cc w/encs: See next page

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