

50-244



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 15, 1995

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas & Electric Corporation  
89 East Avenue  
Rochester, NY 14649

SUBJECT: R. E. GINNA NUCLEAR POWER PLANT - STEAM GENERATOR REPLACEMENT -  
CONCURRENCE ON LICENSEE'S PLANNED REEVALUATION OF THE POSTULATED  
EFFECTS ON THE REACTOR VESSEL INTERNALS (TAC NO. M90043)

Dear Dr. Mecredy:

Rochester Gas & Electric Corporation (RG&E) plans to reduce full-load temperature ( $T_{av}$ ) in the reactor coolant loop from the present 573.5 °F to as low as 559 °F at the Ginna Nuclear Power Plant, following steam generator (SG) replacement in 1996. This reduced coolant temperature results in a denser fluid and for purposes of post-loss-of-coolant accident inertial loading effects, is nonconservative. RG&E, therefore, plans to perform a reevaluation of the effects of a postulated pipe rupture on the reactor vessel internals due to inertial loads.

The consequences from this pipe break are considered to be the most severe in comparison with pipe rupture effects from any other pipe break in a piping system connected to the reactor coolant system (RCS) which has not had an NRC-approved "leak-before-break" safety evaluation. In accordance with GDC-4, a double-ended RCS break is a dynamic effect and may be excluded from the Ginna plant design basis because the analyses submitted to the NRC, and subsequently approved (References 2, 3, 4, and 5), demonstrated that the probability of a rupture in the RCS piping, pressurizer surge line, and the accumulator lines is extremely low under conditions consistent with the design basis for the piping..

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The NRC staff finds this analytical approach, as proposed Reference 1 in the enclosed list of references acceptable, and may at its discretion review the analysis related to the pipe break in the 10" residual heat removal line when it is submitted for information under 10 CFR 50.59 as part of the SG replacement documentation.

Sincerely,

Original signed by  
Allen R. Johnson, Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: As stated

cc w/enclosure:  
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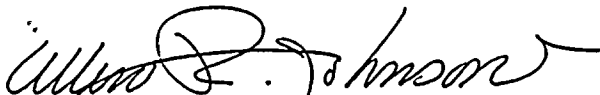
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Allen R. Johnson, Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
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See next page

Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

cc:

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## REFERENCES

1. Letter from Rochester Gas & Electric Corporation (RG&E) to the NRC seeking concurrence on analytical pipe break approach due to reduction in RCS  $T_{avg}$  dated August 2, 1994.
2. Letter from D. Eisenhut (NRC) to All Operating PWR Licensees, Subject: Safety Evaluation of Westinghouse Topical Reports Dealing with Elimination of Postulated Pipe Breaks in PWR Primary Main Loops (Generic Letter 84-04) dated February 1, 1984.
3. Letter from D. DiIanni (NRC) to R. Kober (RG&E), Subject: GL 84-04, dated September 9, 1986.
4. Letter from F. Miraglia (NRC) to All Operating Licensees, Subject: Relaxation in Arbitrary Intermediate Pipe Rupture Requirements (GL 87-11), dated June 19, 1987.
5. Letter from D. Crutchfield (NRC) to John E. Maier (RG&E), Subject: IPSAR Section 4.13, Effects of Pipe Breaks on Structures, Systems, and Components Inside Containment for the R. E. Ginna Nuclear Power Plant, dated June 28, 1983.