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 MECREY, R.C. Rochester Gas & Electric Corp.
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 JOHNSON, A.R. Project Directorate I-3

SUBJECT: Provides listed info on plans to implement seismic aspects of GL-88-20, Suppl 4 re Reactor Coolant Sys structural integrity.

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 TITLE: Generic Ltr 88-20 re Individual Plant Evaluations

NOTES: License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

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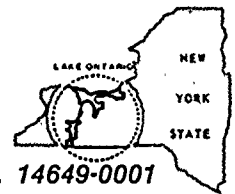
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ROBERT C. MECREDDY
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December 28, 1993

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U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Generic Letter 88-20, Supp. 4
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

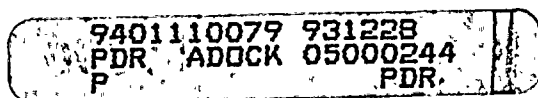
Ref. (a): Letter from A. Johnson (NRC) to R. Mecreddy (RG&E), dated August 11, 1993, Subject: Review of Response to Generic Letter 88-20, Supplement 4 - Individual Plant Examinations for External Events (TAC No. M83624)

Dear Mr. Johnson:

In order to facilitate future discussions, RG&E is providing the following information to NRC relative to our plans to implement the seismic aspects of Generic Letter 88-20, Supp. 4.

1. We are in the process of determining if the Reactor Coolant System is able to maintain its structural integrity in the event of a 0.3g Review Level Earthquake (RLE). Preliminary indications are that such a conclusion will be reached. In that event, a seismically-induced small break LOCA will not be considered a credible event.
2. Using the screening criteria of EPRI NP-6041-SL Rev. 1 and NUREG/CR-4334, we have determined that the containment and internal structures are able to withstand a 0.3g RLE.
3. Based on our PRA being developed in response to Generic Letter 88-20, the frequency of a small break LOCA occurrence is less than 1.0×10^{-2} /reactor year.
4. Based on the EPRI seismic hazard curves, the frequency of a $\geq 0.3g$ earthquake (RLE) is $\approx 5.5 \times 10^{-6}$ /year. This is expected to be very similar to the results from the revised Lawrence Livermore Laboratories seismic hazard curves.

Anticipated Conclusion: The probability of an RLE and a SBLOCA in the same year is less than 5×10^{-8} . Since this is below the screening criteria of NUREG-1407 for reporting, we have determined that the sequence of events which would require evaluation of SBLOCA mitigation equipment should not be evaluated in our GL 88-20



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Supp: 4 response. The containment structure is also capable of withstanding the 0.3g RLE and its evaluation need not be performed for IPEEE.

Other safety-related equipment required to attain and maintain safe shutdown will be evaluated during RG&E's SQUG walkdowns as part of our resolution to USI A-46. In addition to the data being provided to resolve USI A-46, RG&E will perform screening walkdowns for components on that Safe Shutdown Equipment List (SSEL), and sort them into two categories: 1) High Confidence Low Probability of Failure (HCLPF) greater than 0.3g and 2) HCLPF less than or equal to 0.3g. In the second category, RG&E will provide the components' seismic margins beyond our currently licensed 0.2g SSE. This will meet the main objective delineated in NUREG-1407: to provide a qualitative understanding of the effects of postulated severe accident scenarios. The SQUG/IPEEE walkdowns will be performed by a team of experts who have the sophisticated engineering judgement necessary to determine the importance and potential consequences of seismically-induced failures on plant safety. Included within their scope of review will be an assessment of non-seismic failures and human actions as they might affect Ginna Station's safe shutdown functions.

Our previously agreed upon schedule remains the same - completion by May 22, 1995. We are not aware of any other open issues regarding our commitments to GL 88-20, Supp. 4. If so, any meetings, conference calls, or correspondence needed to resolve remaining IPEEE issues should be scheduled expeditiously in order to maintain this schedule.

Very truly yours,


Robert C. Mecredy

GJW\309
Attachment

xc: Mr. Allen R. Johnson (Mail Stop 14D1)
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Ginna Senior Resident Inspector

