



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

February 24, 1993

Docket No. 50-244

Dr. Robert C. Mecredy  
Vice President, Nuclear Production  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: EMERGENCY RESPONSE CAPABILITY - CONFORMANCE TO REGULATORY GUIDE  
1.97, REVISION 3 (TAC NO. M80439)

The NRC issued a safety evaluation (SE) with an attached Technical Evaluation Report (TER) on December 4, 1990, after having performed a review of your conformance to Regulatory Guide (RG) 1.97, Revision 3. The SE/TER found your design acceptable with conformance to RG 1.97, except for instrumentation for monitoring neutron flux, containment isolation valve position, residual heat removal (RHR) heat exchanger outlet temperature, accumulator tank level and pressure, and emergency ventilation damper position.

Two NRC requests for additional information (RAIs), dated March 22, 1991, and July 7, 1992, indicated the need for clarification with regard to Ginna Station's conformance to RG 1.97, Revision 3. Based on discussions with you we determined that there was a misunderstanding in the interpretation of RG 1.97 and you were requested to submit new information. This information was submitted by letters dated May 6 and 16, 1991, June 17, 1991, March 13, 1992, May 8, 1992, and October 14, 1992. Additional information was also submitted in an NRC meeting with you on September 16, 1992, at NRC Headquarters. The meeting included additional clarifications with regard to your conformance to RG 1.97, Revision 3. Commitments and conclusions were published in an NRC meeting summary dated November 24, 1992.

Enclosure 1 transmits to you a supplemental safety evaluation (SSE) with a TER (EGG-RTAP-10240) by Idaho National Engineering Laboratory (INEL) (Attachment 1). We have reviewed the evaluation performed by INEL contained in the TER and concur with its bases and findings. You conform to RG 1.97 guidance, or have provided an acceptable justification for deviations from RG 1.97 guidance, except for instrumentation associated with post-accident neutron flux monitoring. Post-accident neutron flux monitoring is not addressed in this SE/TER, but will be addressed in a forthcoming separate SSE.

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Enclosure 2 provides a list of references.

Sincerely,

/s/

Allen R. Johnson, Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. NRR SSE w/attached INEL TER
- 2. List of References

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Dr. Robert C. Mecredy

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February 24, 1993

Enclosure 2 provides a list of references.

Sincerely,



Allen R. Johnson, Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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2. List of References

cc w/enclosures:  
See next page



Dr. Robert C. Mecredy

R.E. Ginna Nuclear Power Plant

cc:

Thomas A. Moslak, Senior Resident Inspector  
R.E. Ginna Plant  
U.S. Nuclear Regulatory Commission  
1503 Lake Road  
Ontario, New York 14519

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

Ms. Donna Ross  
Division of Policy Analysis & Planning  
New York State Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

Charlie Donaldson, Esq.  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, New York 10271

Nicholas S. Reynolds  
Winston & Strawn  
1400 L St. N.W.  
Washington, DC 20005-3502

Ms. Thelma Wideman  
Director, Wayne County Emergency  
Management Office  
Wayne County Emergency Operations Center  
7370 Route 31  
Lyons, New York 14489

Ms. Mary Louise Meisenzahl  
Administrator, Monroe County  
Office of Emergency Preparedness  
111 West Fall Road, Room 11  
Rochester, New York 14620