JUL 21 1992

Docket No. 50-244

Dr. Robert C. Mecredy
Vice President
Ginna Nuclear Production
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: INSPECTION REPORT NO. 50-244/92-02

This letter supplements our letter of June 15, 1992, regarding the NRC staff review of the additional information provided in the RG&E letter dated May 4, 1992, for Violation C and Example 5 to Violation A, identified in the subject inspection report.

With regards to Violation C, the staff concluded that your response does not adequately address the issue for which the violation was written. Specifically, RG&E did not substantiate that pre-operational test results were reviewed **prior to** making changes to the service water system (SWS) configuration and operation to determine if additional testing of the modified system would be necessary to support its original design capabilities. Since the Ginna Technical Specifications requires at least two service water pumps to be operable, a single active failure could result in only one service water pump being available during the accident condition. Additionally, if the LOCA occurs without a loss of off-site power, the non-safety-related (NSR) loads would not be isolated. Therefore, the SWS must be able to remove heat from safety-related components during both the injection and recirculation phases of a LOCA with only one service water pump operating while also supplying NSR loads. This capability had not been demonstrated. Therefore, this violation stands as written. The NRC staff is also evaluating whether common mode passive failures such as pipe crack/break scenarios should have been evaluated for the cross-connected configuration.

Regarding the information provided by RG&E for disagreeing that Example 5 to Violation was an appropriate example for the stated violation, the staff does not concur with RG&E's assessment. Your response does not substantiate that using an inlet service water temperature of 75°F in the design analysis for establishing a minimum diesel generator jacket cooler and lube oil cooler service water flow requirements is more conservative than using a service water temperature of 80°F.

Please recognize that much of the information contained in your letter is based on recent analysis by RG&E in response to NRC concerns and is considered corrective action. Accordingly, it cannot be credited for refuting a violation.

OFFICIAL RECORD COPY

IE01

Although no further response to this letter is required, it is our expectation that you will take actions to address the concerns reiterated in this letter. Your corrective actions to these violations, as well as your overall actions to avoid further violations, will be reviewed during a future inspection of your licensed program.

Your cooperation with us in this matter is appreciated.

Sincerely,

ORIGINAL SIGNED BY:
Charles W. Hehl, Director
Division of Reactor Projects

cc:
R. Smith, Senior Vice President, Production and Engineering
Central Records (4 copies)
Director, Power Division
State of New York, Department of Law (w/cy of Licensee's Response Letter)
N. Reynolds, Esquire (w/cy of Licensee's Response Letter)
Public Document Room (PDR) (w/cy of Licensee's Response Letter)
Local Public Document Room (LPDR) (w/cy of Licensee's Response Letter)
Nuclear Safety Information Center (NSIC) (w/cy of Licensee's Response Letter)
NRC Resident Inspector (w/cy of Licensee's Response Letter)
State of New York, SLO Designee (w/cy of Licensee's Response Letter)

bcc w/cy of Licensee's Response Letter:
W. Lazarus, DRP
J. Linville, DRP
A. Johnson, PM, NRR
T. Moslak, SRI - Ginna

RI:DRP	RI:DRP	RI:DRP	RI:DRP	RI:DRP
Knutson/meo	Moslak	Lazarus	Linville	Hehl
7/11/92	7/6/92	7/6/92	7/15/92	7/21/92

OFFICIAL RECORD COPY
A:REP9202A.MEO