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ACCESSION NBR: 9203130310 DOC.DATE: 77/12/16 NOTARIZED: NO DOCKET #
FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
AUTH.NAME AUTHOR AFFILIATION
CASE, E.G. Office of Nuclear Reactor Regulation, Director
RECIP.NAME RECIPIENT AFFILIATION
WHITE, L.D. Rochester Gas & Electric Corp.

SUBJECT: Forwards "SER on ECCS Evaluation Model for Westinghouse Two-Loop Plants." for Nov 1977.

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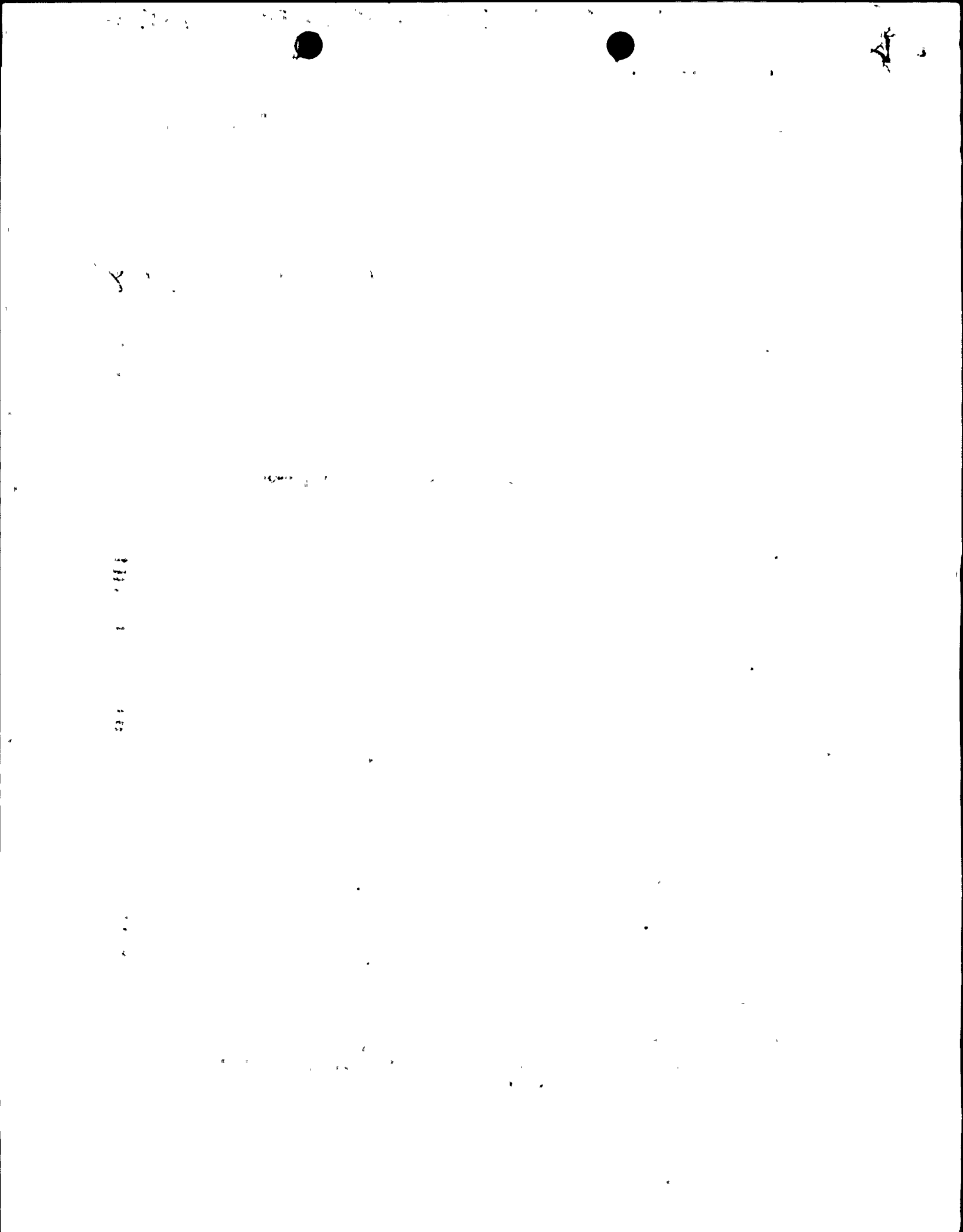
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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DEC 19 1977

December 16, 1977

Docket No. 50-244

Rochester Gas and Electric Corporation
ATTN: Mr. Leon D. White, Jr.
Vice President - Electric and
Steam Production
89 East Avenue
Rochester, New York 14604

See Mar. 15, 1978

Distr. to Ginna
NSARB

Response instructions:

John Arthur to
prepare

Gentlemen:

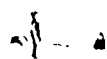
As you know, we have been re-evaluating the acceptability of the calculational model used to evaluate the performance of the emergency core cooling system (ECCS) in Westinghouse designed two reactor coolant loop plants, such as your R. E. Ginna Atomic Power Station. Results of our evaluation of the model are presented in our Safety Evaluation Report on ECCS Evaluation Model for Westinghouse Two-Loop Plants, a copy of which is provided as Enclosure 1. This evaluation concludes that the presently approved model does not appropriately account for ECCS water injected above the core.

In addition, we have performed analyses to determine the immediate safety significance of this conclusion for operating two-loop facilities and the nature and timing of any corrective action that may be needed. A discussion of these analyses and our conclusions are presented in our Safety Evaluation Report on Continued Safe Operation of Westinghouse Designed Two-Loop Plants, a copy of which is provided as Enclosure 2. We have concluded that operation of your two-loop facility may continue safely for a limited period of time while we determine, after discussions with you, the proper application of the staff re-evaluation of the Westinghouse two-loop model to your plant.

Accordingly, interim bases for continued safe operation of your facility must be developed within the next 30 days, taking into account the apparent deficiencies in the Westinghouse two-loop models described in our Safety Evaluation Reports. We believe that such interim bases are likely to involve some additional operating limits to compensate for these model deficiencies. In addition, a permanent resolution of the problems giving rise to these model deficiencies should be developed and provided to us, along with a schedule for its implementation, as soon as possible, but not later than 60 days after this 30 day period.

DFOI
per: Ramin Assa
3/13/92

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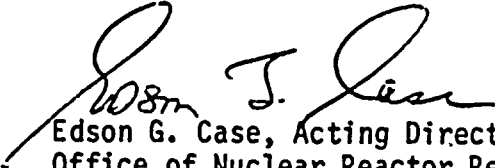
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December 16, 1977

Therefore, pursuant to 10 CFR 50.54(f) of the Commission's regulations, you are requested to propose, within 30 days from the date of this letter, appropriate bases, including any necessary operating limitations, to justify continued operation of your facility beyond this 30 day period. Any subsequent action that may be required will be based on our evaluation of your submittals. If you do not choose to propose alternative bases, the staff will prepare suitable operating limitations based on its reassessment.

We will be happy to meet with you or any of your representatives to discuss this matter. Please contact your NRC Project Manager if you wish such a meeting or if you have any questions.

Sincerely,


Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation Report
on ECCS Evaluation Model
for Westinghouse Two-Loop
Plants
2. Safety Evaluation Report on
Continued Safe Operation
of Westinghouse Designed
Two-Loop Plants

cc w/enclosures:
See next page

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