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AUTH.NAME AUTHOR AFFILIATION
MECREDY,R.C. Rochester Gas & Electric Corp.
RECIP.NAME RECIPIENT AFFILIATION
JOHNSON,A.R. Project Directorate I-3

SUBJECT: Forwards revised proposed FOL DPR-18 to be substituted in entirety for Exhibit A attached to 901005 license amend application,changing expiration date from 20060425 to 20090918.

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NOTES:License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

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ROBERT C. MECREDY
Vice President
Ginna Nuclear Production

TELEPHONE
AREA CODE 716 546-2700

December 6, 1990

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Change in Expiration of Ginna Operating License

Reference: Amendment No. 26 to FOL No. DPR-18
dated April 7, 1988

Dear Mr. Johnson:

By letter dated October 5, 1989, RG&E requested that a License Amendment be issued to reflect a change in expiration of the Ginna Operating License from April 25, 2006 to September 18, 2009. Section 2.E of Exhibit A of that submittal, entitled "Physical Protection," inadvertently did not include provisions contained in Amendment No. 26 to FOL No. DPR-18, dated April 7, 1988 which made changes to Section 2.E.

Attached is a revised proposed Facility Operating License (Exhibit A) which should be substituted in its entirety for Exhibit A attached to our Oct. 5, 1989 submittal.

No additional Safety Evaluation for the change to Section 2.E is considered necessary, since this is merely an administrative change made to reflect previously approved NRC documentation.

Very truly yours,

Robert C. Mecredy

Robert C. Mecredy

Attachment

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xc: Mr. Allen Johnson (Mail Stop 14D1)
Project Directorate I-3
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Ginna Senior Resident Inspector

EXIBIT A

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R.E. GINNA NUCLEAR POWER PLANT

FACILITY OPERATING LICENSE

License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application complies with the requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;

   B. Construction of the R.E. Ginna Nuclear Power Plant (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-19, as amended, and the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   D. There is reasonable assurance (i) that the facility can be operated at power levels up to 1520 megawatts (thermal) without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the regulations of the Commission;

   E. The applicant is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;

   F. The applicant has furnished proof of financial protection that satisfies the requirements of 10 CFR Part 140; and

   G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public.

2. The Provisional Operating License dated September 19, 1969, is superseded by Facility Operating License No. DPR-18 hereby issued to Rochester Gas and Electric Corporation to read as follows:

A. The license applies to the R.E. Ginna Nuclear Power Plant, a closed cycle, pressurized, light-water-moderated and cooled reactor, and electric generating equipment (herein referred to as "the facility") which is owned by the Rochester Gas and Electric Corporation (hereinafter "the licensee" or "RG&E"). The facility is located on the licensee's site on the south shore of Lake Ontario, Wayne County, New York, about 16 miles east of the City of Rochester and is described in license application Amendment No. 6, "Final Facility Description and Safety Analysis Report", and subsequent amendments thereto, and in the application for power increase notarized February 2, 1971, and Amendment Nos. 1 through 4 thereto (herein collectively referred to as "the application").

B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses RG&E:

(1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities", to possess, use, and operate the facility at the designated location in Wayne County, New York, in accordance with the procedures and limitations set forth in this license;

(2) Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material or reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation as described in the Final Safety Analysis Report, as amended, and Commission Safety Evaluations dated November 15, 1976, October 5, 1984, and November 14, 1984.

(a) Pursuant to the Act and 10 CFR Part 70, to receive and store four (4) mixed oxide fuel assemblies in accordance with the licensee's application dated December 14, 1979 (transmitted by letter dated December 20, 1979);

(b) Pursuant to the Act and 10 CFR Part 70, to possess and use four (4) mixed oxide fuel assemblies in accordance with the licensee's application dated December 14, 1979 transmitted by letter dated December 20, 1979, as supplemented February 20, 1980 and March 5, 1980;

(3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

(1) Maximum Power Level

RG&E is authorized to operate the facility at steady-state power levels up to a maximum of 1520 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

(a) The licensee shall maintain in effect all fire protection features described in the licensee's submittals referenced in and as approved or modified by the NRC' Fire Protection Safety Evaluation (SE) dated February 14, 1979 and SE supplements dated December 17, 1980, February 6, 1981 and June 22, 1981, or configurations subsequently approved by the NRC, subject to provisions (b) and (c) below.

(b) The licensee may make no change to the approved fire protection features which would decrease the level of fire protection in the plant without prior approval of the Commission. To make such a change the licensee must submit an application for license amendment pursuant to 10 CFR 50.90.

(c) The licensee may make changes to approved fire protection features which do not decrease the level of fire protection without prior Commission approval provided such changes do not otherwise involve a change in a license condition or technical specification or result in an unreviewed safety question (see 10 CFR 50.59). However, the licensee shall maintain, in an auditable form, a current record of all such changes including an anlaysis of the effects of the change on the level of fire protection and shall make such records available to NRC inspectors upon request. All changes to the approved features made without prior Commission approval shall be reported annually to the Director of the Office of Nuclear Reactor Regulation.

(4) Secondary Water Chemistry Monitoring Program

The licensee shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall be described in the plant procedures and shall incldue:

(a) Identification of a sampling schedule for the critical parameters and control points for these parameters;

(b) Identification of the procedures used to measure the values of the critical parameters;

(c) Identification of process sampling points;

(d) Procedure for the recording and management of data;

(e) Procedures defining corrective actions for off control point chemistry conditions; and

(f) A procedure identifying (i) the authority responsible for the interpretation of the data, and (ii) the sequence and timing of administrative events required to initiate corrective action.

(5) Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as reasonably achievable levels. This program shall include the following:

(a) Provisions establishing preventive maintenance and periodic visual inspection requirements; and

(b) Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

(6) Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

(a) Training for personnel;

(b) Procedures for monitoring; and

(c) Provisions for maintenance of sampling and analysis equipment.

D. The facility requires exemptions from certain requirements of 10 CFR 50.46(a)(1), 50.48(c)(4), and Appendix J to 10 CFR Part 50. These include: (1) an exemption from 50.46(a)(1), that ECCS performance be calculated in accordance with an acceptable calculational model which conforms to the provisions in Appendix K (SER dated April 18, 1978). The exemption will expire upon receipt and approval of revised ECCS calculations; (2) certain exemptions from Appendix J to 10 CFR Part 50 section III.A.4(a) maximum allowable leakage rate for reduced pressure tests, section III.B.1 acceptable technique for performing local (Type B) leakage rate tests, section III.D.1 scheduling of containment integrated leakage rate tests, and section III.D.2 testing interval for containment airlocks (SER dated March 28, 1978); and (3) an exemption to the schedular requirements for the alternative shutdown system as set forth in 10 CFR 50.48(c)(4) (NRC letter dated May 10, 1984). The exemption is effective until startup from the 1986 refueling outage. The aforementioned exemptions ar authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, the exemptions are hereby granted pursuant to 10 CFR 50.12.

E. Physical Protection - The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27827 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Robert Emmet

Ginna Nuclear Plant Physical Security Plan," with revisions submitted through August 18, 1987; "Robert Emmet Ginna Nuclear Plant Guard Training and Qualification Plan" with revisions submitted through July 30, 1981; and "Robert Emmet Ginna Nuclear Plant Safeguards Contingency Plan" with revisions submitted through April 14, 1981. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

F. The amended license is effective as of the date of issuance and shall expire at midnight, September 18, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION

Attachment:
Appendix A - Technical Specifications

Date of Issuance: