

# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### ROCHESTER GAS AND ELECTRIC CORPORATION

#### DOCKET NO. 50-244

## R. E. GINNA NUCLEAR POWER PLANT

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39 License No. DPR-18

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated February 16, 1990, as supplemented on March 15, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the . Commission:
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

## (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 39, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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FOR THE NUCLEAR REGULATORY COMMISSION

for

Patrick M. Sears

Richard H. Wessman, Director Project Directorate I-3 Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: April 12, 1990

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FOR THE NUCLEAR REGULATORY COMMISSION

Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance:

April 12, 1990

# ATTACHMENT TO LICENSE AMENDMENT NO. 39

## FACILITY OPERATING LICENSE NO. DPR-18

## **DOCKET NO. 50-244**

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

 $\frac{\text{Remove}}{5.3-1}$ 

<u>Insert</u> 5. 3-1

## 5.3 Reactor Design Features

## 5.3.1 Reactor Core

The reactor core contains approximately 45 metric tons of uranium in the form of uranium dioxide pellets. The pellets are encapsulated in Zircaloy 4 tubing to form The reactor core is made up of 121 fuel fuel rods. assemblies" with each fuel assembly containing 179 Fuel rod locations at any time fuel rod locations: during plant life, may consist of fuel rods clad with Zircaloy -4 or filler rods fabricated from Zircaloy -4 or stainless steel if justified by cycle-specific reload analysis. Should more than 30 rods in the core. or 10 rods in any assembly be replaced per refueling, a report describing the number of rods replaced and associated cycle-specific evaluation shall be submitted to the Commission prior to criticality.

Each fuel assembly also contains 16 guide tubes and one instrumentation thimble all arranged in a  $14 \times 14$  array to form a fuel assembly.

- b. The enrichment of reload fuel shall be no more than 3.5 weight per cent U-235 for regions delivered prior to January 1, 1984 (Regions 1-15), 4.25 weight per cent U-235 for regions delivered after January 1, 1984, or their equivalents in terms of reactivity.
- c. There are 29 full-length assemblies in the reactor core. Each RCC assembly contains 16 144 inch lengths of silver-indium-cadmium alloy clad with stainless steel which act as neutron absorbers when inserted into the core. (5)

### Basis

The DNBRs for the reconstituted assemblies are conservatively determined by assuming the filler rods are operating at the highest power in the reconstituted fuel assembly.