



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS & ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 32
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas & Electric Corporation (the licensee), dated October 10, 1988, and supplemented December 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.B(6)(b) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(b) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 32, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 8, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 32

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by the captioned amendment number and contains marginal lines indicating the area of change.

REMOVE

6.1-1
6.2-1
6.4-1
6.5-1
6.5-2
6.5-3
6.5-4

6.5-10
6.5-11
6.6-1
6.7-1
6.8-1
6.8-2
6.13-2

INSERT

6.1-1
6.2-1
6.4-1
6.5-1
6.5-2
6.5-3
6.5-4
6.5-4a*
6.5-10
6.5-11
6.6-1
6.7-1
6.8-1
6.8-2
6.13-2

*Denotes new page

6.0

ADMINISTRATIVE CONTROLS

6.1

RESPONSIBILITY

6.1.1

The Plant Manager, Ginna Station shall be responsible for overall on-site Ginna Station operation and shall delegate in writing the succession to this responsibility during his absence.

Onsite and Offsite Organization

An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all Plant management positions. Those relationships shall be documented and updated, as appropriate, in the form of organization charts. These organization charts will be documented in the UFSAR and updated in accordance with 10CFR50.71.

b. The Vice President, Production and Engineering, shall have corporate responsibility for overall Plant nuclear safety, and shall take any measures needed to assure acceptable performance of the staff in operating, maintaining, and providing technical support in the Plant so that continued nuclear safety is assured.

c. The Plant Manager, Ginna Station shall have responsibility for overall unit operation and shall have control over those resources necessary for safe operation and maintenance of the Plant.

6.4

TRAINING

6.4.1

A retraining and replacement training program for the facility staff shall be maintained under the direction of the Division Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10.CFR Part 55.

6.4.2

The training program shall meet or exceed NFPA No. 27, 1975 Section 40, except that (1) training for salvage operations need not be provided and (2) the Fire Brigade training sessions shall be held at least quarterly. Drills are considered to be training sessions.

REVIEW AND AUDIT

Three separate organizational units shall be established for the purpose of review and audit of plant operations and safety-related matters. One of these will be an on-site operations review group, the Plant Operations Review Committee (PORC). A second is the Quality Assurance (Q.A.) group, responsible for the audit of safety-related activities associated with plant operations. A third is the independent audit and review group, the Nuclear Safety Audit and Review Board (NSARB). This group is responsible for the periodic review of the activities of the Plant Operations Review Committee, for directing audits and evaluating their results, and for the management evaluation of the status and adequacy of the Quality Assurance program.

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC) FUNCTION

6.5.1.1 The Plant Operations Review Committee shall function to advise the Plant Manager, Ginna Station on all matters related to nuclear safety and for referral of appropriate matters to the Nuclear Safety Audit and Review Board.

COMPOSITION

6.5.1.2 The PORC shall be composed of the;

Chairman: Superintendent, Ginna Production

Vice Chairman/Member: Superintendent, Ginna Support Services

Vice Chairman/Member: Operations Manager

Member: Maintenance Manager

Member: Maintenance Planning/Scheduling Manager

Member: Instrument & Control Supervisor

Member: Technical Manager

Member: Results and Test Supervisor

Member: Reactor Engineer

Member: Health Physics and Chemistry Manager

Member: Nuclear Assurance Manager

Member: Quality Control Engineer

ALTERNATES

6.5.1.3 Alternate members shall be designated by name, in writing by the Chairman.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman.

QUORUM

6.5.1.5 A quorum of the PORC shall consist of the Chairman or Vice Chairman and four members including alternates. No more than two shall be alternates.

RESPONSIBILITIES

.6.5.1.6 The PORC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Manager, Ginna Station to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President, Production and Engineering, and to the Chairman of the Nuclear Safety Audit and Review Board.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Safety Audit and Review Board.

RESPONSIBILITIES (Continued)

- h. Review of the Plant Security Plan and shall submit recommended changes to the Chairman of the Nuclear Safety Audit and Review Board.
- i. Review of the Radiation Emergency Plan and shall submit recommended changes to the Chairman of the Nuclear Safety Audit and Review Board.
- j. Review of implementing procedures for the Plant Security Plan and the Radiation Emergency Plan and proposed changes thereto.
- k. Review of all Reportable Events.

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend in writing to the Plant Manager, Ginna Station approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (d) above constitutes an unreviewed safety question as defined in 10 CFR Section 50.59.

- c. Provide immediate written notification to the Vice President, Production and Engineering, and the Nuclear Safety Audit and Review Board of disagreement between the PORC and the Plant Manager, Ginna Station; however, the Plant Manager, Ginna Station shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8

The PORC shall maintain written minutes of each meeting and copies shall be provided to the Vice President, Production and Engineering, the Chairman of the Nuclear Safety Audit and Review Board, and such others as the Chairman may designate.

AUDITS (Continued)

- g. The Facility Fire Protection Program and implementing procedures at least once per two years.
- h. An independent fire protection and loss prevention program inspection and audit performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- i. An inspection and audit of the fire protection and loss prevention program performed by non-licensee personnel at least once per 36 months. The personnel may be representatives of ANI, an insurance brokerage firm, or other qualified individuals.
- j. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- k. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.
- l. The Process Control Program and implementing procedures at least once per 24 months.
- m. Any other area of facility operation considered appropriate by the NSARB or the Vice President, Production and Engineering.

AUTHORITY

- 6.5.2.9
- a. The Chairman of the Nuclear Safety Audit and Review Board is responsible to the President on all activities for which the Review Board is responsible.
 - b. The NSARB shall report to and advise the Vice President, Production and Engineering, on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

- 6.5.2.10 Records of NSARB activities shall be prepared, approved, and distributed as indicated below:
- a. Minutes shall be recorded of all meetings of this Board. Copies of the minutes shall be forwarded within 14 days following each meeting to the Corporate Chairman of the Board, Vice President, Production and Engineering and such others as the Chairman of the NSARB may designate.
 - b. Reports of reviews encompassed by Section 6.5.2.7 e,f,g and h above, shall be prepared, approved and forwarded to the Vice President, Production and Engineering within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President, Production and Engineering and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6.1

The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each Reportable Event shall be reviewed by the PORC and the results of this review shall be submitted to the NSARB and the Vice President, Production and Engineering.



Country	1950	1960	1970	1980	1990	2000	2010	2020	2030	2040	2050
Japan	7.0	8.0	9.0	10.0	11.0	12.0	13.0	14.0	15.0	16.0	17.0
Germany	10.0	11.0	12.0	13.0	14.0	15.0	16.0	17.0	18.0	19.0	20.0
France	11.0	12.0	13.0	14.0	15.0	16.0	17.0	18.0	19.0	20.0	21.0
Italy	12.0	13.0	14.0	15.0	16.0	17.0	18.0	19.0	20.0	21.0	22.0
Spain	13.0	14.0	15.0	16.0	17.0	18.0	19.0	20.0	21.0	22.0	23.0
Sweden	14.0	15.0	16.0	17.0	18.0	19.0	20.0	21.0	22.0	23.0	24.0
Belgium	15.0	16.0	17.0	18.0	19.0	20.0	21.0	22.0	23.0	24.0	25.0
United Kingdom	16.0	17.0	18.0	19.0	20.0	21.0	22.0	23.0	24.0	25.0	26.0
United States	17.0	18.0	19.0	20.0	21.0	22.0	23.0	24.0	25.0	26.0	27.0
Canada	18.0	19.0	20.0	21.0	22.0	23.0	24.0	25.0	26.0	27.0	28.0
Australia	19.0	20.0	21.0	22.0	23.0	24.0	25.0	26.0	27.0	28.0	29.0
South Africa	20.0	21.0	22.0	23.0	24.0	25.0	26.0	27.0	28.0	29.0	30.0
India	21.0	22.0	23.0	24.0	25.0	26.0	27.0	28.0	29.0	30.0	31.0
China	22.0	23.0	24.0	25.0	26.0	27.0	28.0	29.0	30.0	31.0	32.0
Indonesia	23.0	24.0	25.0	26.0	27.0	28.0	29.0	30.0	31.0	32.0	33.0
Philippines	24.0	25.0	26.0	27.0	28.0	29.0	30.0	31.0	32.0	33.0	34.0
Thailand	25.0	26.0	27.0	28.0	29.0	30.0	31.0	32.0	33.0	34.0	35.0
Malaysia	26.0	27.0	28.0	29.0	30.0	31.0	32.0	33.0	34.0	35.0	36.0
Singapore	27.0	28.0	29.0	30.0	31.0	32.0	33.0	34.0	35.0	36.0	37.0
South Korea	28.0	29.0	30.0	31.0	32.0	33.0	34.0	35.0	36.0	37.0	38.0
Taiwan	29.0	30.0	31.0	32.0	33.0	34.0	35.0	36.0	37.0	38.0	39.0
Hong Kong	30.0	31.0	32.0	33.0	34.0	35.0	36.0	37.0	38.0	39.0	40.0
China (excl. HK)	31.0	32.0	33.0	34.0	35.0	36.0	37.0	38.0	39.0	40.0	41.0
India (excl. HK)	32.0	33.0	34.0	35.0	36.0	37.0	38.0	39.0	40.0	41.0	42.0
Indonesia (excl. HK)	33.0	34.0	35.0	36.0	37.0	38.0	39.0	40.0	41.0	42.0	43.0
Philippines (excl. HK)	34.0	35.0	36.0	37.0	38.0	39.0	40.0	41.0	42.0	43.0	44.0
Thailand (excl. HK)	35.0	36.0	37.0	38.0	39.0	40.0	41.0	42.0	43.0	44.0	45.0
Malaysia (excl. HK)	36.0	37.0	38.0	39.0	40.0	41.0	42.0	43.0	44.0	45.0	46.0
Singapore (excl. HK)	37.0	38.0	39.0	40.0	41.0	42.0	43.0	44.0	45.0	46.0	47.0
South Korea (excl. HK)	38.0	39.0	40.0	41.0	42.0	43.0	44.0	45.0	46.0	47.0	48.0
Taiwan (excl. HK)	39.0	40.0	41.0	42.0	43.0	44.					

SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR Section 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Vice President, Production and Engineering, to the NSARB and to the Commission immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NSARB, and the Vice President, Production and Engineering within two weeks of the violation.

6.8

PROCEDURES

6.8.1

Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. The radiological environmental monitoring program.
- h. Offsite Dose Calculation Manual implementation.
- i. Process Control Program implementation.

6.8.2

Each procedure and administrative policy of 6.8.1 above, the changes thereto, shall be reviewed by the PORC and approved by the Plant Manager, Ginna Station prior to implementation and reviewed periodically as set forth in the applicable procedures.

6.8.3

Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedures is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom is the Shift Supervisor who holds a Senior Reactor Operator's License.

c. The change is documented, reviewed by the PORC, and approved by the Plant Manager, Ginna Station within 10 days of implementation.

radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and who will perform periodic radiation surveillance at the frequency specified in the HPWP. The surveillance frequency will be established by a plant Health Physicist.

- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1 a. above, and in addition locked doors shall be provided to prevent unauthorized entry into these areas and the keys to unlock these locked doors shall be maintained under the administrative control of the Shift Supervisor on duty.