NRC FORM 366 (7-77)

79/011/01T-0

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

(CAR 1223)

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
0 1	LICENSEE CODE (2)	N Y R E G 1
	LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
	LICENSE TYPE (4)	4 1 1 1 1
	CAT (5)	
CON'T 0 1	REPORT SOURCE (6)	L
	DOCKET NUMBER (7)	0 5 0 0 0 2 4 4
	EVENT DATE (8)	0 5 1 1 7 9
	REPORT DATE (9)	0 5 2 5 7 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Licensee was notified by the manufacturer of the pressurizer power operated relief
0 3 valve that the template used to fabricate the valve internals was not within required
0 4 design specifications. These valves may not meet the required relieving capacity as
0 5 delineated in the RCS overpressure protection system design. (T.S. 6.9.2.a.(9)) As
0 6 RCS was at power conditions, the overpressure protection system was not required to
0 7 be operable.
0 8

0 9	SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
	C J	B	B	V A L V E X	F	B

LER/RO REPORT NUMBER (17)	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 9	—	0 1 1	/	0 1	T	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
Z	A	Z	Z	0 0 0 0	Y	N	N	C 6 3 5

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Error in tracer lathe template. Manufacturer is expediting delivery of properly sized
1 1 internals, which, after receipt of parts, will be installed next cold shutdown that
1 2 valves can be isolated. Valves are Copes Vulcan Inc. 2" 1500 psi ASA standard with
1 3 3" inlet and outlet, valve identification No. 3ID58RP model no. 132349, operating
1 4 mode reverse.

1 5	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
	E	1 0 0	NA	D	Notification from manufacturer

1 6	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z	Z	NA	NA

1 7	PERSONNEL EXPOSURES NUMBER (37)	TYPE (38)	DESCRIPTION (39)
	0 0 0	Z	NA

1 8	PERSONNEL INJURIES NUMBER (40)	DESCRIPTION (41)
	0 0 0	NA

1 9	LOSS OF OR DAMAGE TO FACILITY TYPE (42)	DESCRIPTION (43)
	Z	NA

2 0	PUBLICITY ISSUED (44)	DESCRIPTION (45)	NRC USE ONLY
	N	NA	

7905310435

NAME OF PREPARER S. Spector PHONE: 716/546-2700, ext.291-215

Attachment to LER 79-011/01T-0
Rochester Gas and Electric Corporation
R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

RG&E was notified by Copes Vulcan of a manufacturing error in the internals for our pressurizer power operated relief valves that reduced the flow coefficient C_v of the valves to 42 versus the design C_v of 50. This design value was used for the design of the RCS overpressure protection system. This error may cause the maximum pressure during a postulated low temperature overpressure transient to exceed the allowable pressures of Technical Specification 3.1.2.

New valve internals that would meet the design relief rate are being expedited by the manufacturer and should be available in June. These internals will be installed as soon as practical.

RG&E does not have access to the necessary information to determine if this error has affected any other plants. We have notified Copes Vulcan of the implications of this error.