

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8106040270 DOC. DATE: 81/05/15 NOTARIZED: NO DOCKET #  
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester, G 05000244  
 AUTH. NAME: AUTHOR AFFILIATION  
 SNOW, B. A. Rochester Gas & Electric Corp.  
 RECIP. NAME: RECIPIENT AFFILIATION  
 GRIER, B. H. Region 1, Philadelphia, Office of the Director (81/03/01)

SUBJECT: RO 81-009; on 810515, during refueling & maint outage, eddy current examinations revealed abnormal degradation of steam generator B inlet tubes. Crevice indication tubes will be sleeved. Hot leg tubes will be removed.

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 TITLE: Incident Reports

NOTES: 1 copy: SEP Sect. Ldr.

05000244

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JUN 05 1981

TOTAL NUMBER OF COPIES REQUIRED: LTTR

63

62 ENCL

63

62

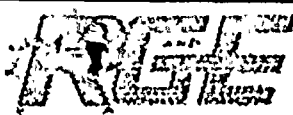
14



1. The first part of the document is a list of names and addresses. The names are written in a cursive script, and the addresses are written in a more formal, printed style. The list is organized into columns, with names in the first column and addresses in the second column. The names are: John Doe, Jane Smith, and Mary White. The addresses are: 123 Main Street, New York, NY 10001; 456 Elm Street, New York, NY 10002; and 789 Oak Street, New York, NY 10003.

2. The second part of the document is a list of names and addresses. The names are written in a cursive script, and the addresses are written in a more formal, printed style. The list is organized into columns, with names in the first column and addresses in the second column. The names are: John Doe, Jane Smith, and Mary White. The addresses are: 123 Main Street, New York, NY 10001; 456 Elm Street, New York, NY 10002; and 789 Oak Street, New York, NY 10003.

100



PDR



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

TELEPHONE  
AREA CODE 716 546-2700

May 15, 1981

50-244

Mr. Boyce H. Grier, Director  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region I  
631 Park Avenue  
King of Prussia, PA 19406

SUBJECT: Reportable Occurrence 81-009, Abnormal Degradation of Steam Generator Tubes

Dear Mr. Grier:

This is confirm my phone call on May 15, 1981 to report Reportable Occurrence 81-009 at Ginna Station. A program of planned Eddy Current examinations was conducted on both Loop "A" and "B" Steam Generators at Ginna Station from May 7th to 12th, 1981 during the present refueling and maintenance outage. This examination consisted of a multi-frequency Eddy Current examination to detect and measure potential tube defects. The inlets of both steam generators received a 100% inspection and the outlets received approximately a 25% inspection.

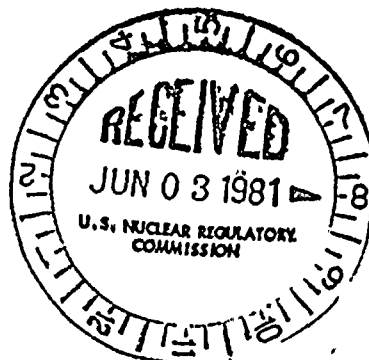
The results of these examinations did not reveal any reportable indications in the outlets of either steam generator and the inlet of the "A" steam generator. The "B" steam generator inlet has 45 tubes with indications from the multi-frequency data of which 44 are crevice indications in the tube sheet crevice area and 1 other indication above the tube sheet out on the periphery. Of these 44 tubes 6 tubes were above our 40% degradation criteria.

Based upon these results steam generator tube sleeving shall commence by sleeving the 44 crevice indication tubes and plugging the 1 tube in the periphery. After tube sleeving operations are complete tube pulling will be initiated to remove 3 tubes from the "B" generator hot leg.

Very truly yours,

*Bruce A. Snow*

Bruce A. Snow  
Plant Superintendent



BAS:lhj

cc: J. E. Maier  
L. S. Lang  
J. C. Noon

A002  
5/11

8106040270  
S



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## LICENSEE EVENT REPORT

**CONTROL BLOCK:**

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

**0** **1**      **(2)**      **(3)**      **(4)**      **(5)**

7 8      9      14      15      25      26      30      57 CAT 58

LICENSEE CODE      LICENSE NUMBER      LICENSE TYPE

**CON'T**

0	1	REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE									
7	8	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80																			
			6	0	5	0	0	0	2	4	4	7	0	5	1	5	8	1	8	0	3	1	5	8	1	9															

### EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	
0	3	
0	4	
0	5	
0	6	
0	7	
0	8	

7 8 9

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE	
0	9												
7	8	9	10	11	12	13	14	15	16	17	18	19	20
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.			
17	21	22	23	24	25	26	27	28	29	30	31	32	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.	
33	34	35	36	37	38	39	40	41	42	43	44	45	46
PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER											

### CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	
1	1	
1	2	
1	3	
1	4	

7 8 9  
FACILITY STATUS (28) % POWER (29) OTHER STATUS (30) METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)  
1 5 7 8 9 10 12 13 44 45 46 80

**ACTIVITY .CONTENT**

**RELEASED OF RELEASE**

**AMOUNT OF ACTIVITY** (35)

**LOCATION OF RELEASE** (36)

PERSONNEL EXPOSURES		TYPE		DESCRIPTION (39)
NUMBER		(37)	(38)	
1	7			

7		8		9		11		12		13	
						PERSONNEL INJURIES					
		NUMBER				DESCRIPTION		(41)			
1	8					(40)					

7		8	9	11	12	
1		9	(42)		LOSS OF OR DAMAGE TO FACILITY (43)	
TYPE		DESCRIPTION				
1	9		(42)			

7 8 9 10

PUBLICITY DESCRIPTION (45)

ISSUED (44)

NRC USE ONLY

68 69

**NRC USE ONLY**

NAME OF PREPARER

**PHONE:**

NRC FORM  
(7-77)