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FACIL:50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
AUTH.NAME AUTHOR AFFILIATION
SNOW,B.A. Rochester Gas & Electric Corp.
RECIP.NAME RECIPIENT AFFILIATION
GRIER,B.H. Region 1, Philadelphia, Office of the Director (81/03/01)

SUBJECT: RO 81-009:on 810515,during refueling & maint outage,eddy current examinations revealed abnormal degradation of steam generator B inlet tubes.Crevice indication tubes will be sleeved.Hot leg tubes will be removed.

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PDR

May 15, 1981

50-244

Mr. Boyce H. Grier, Director
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, PA 19406

SUBJECT: Reportable Occurrence 81-009, Abnormal Degradation of Steam Generator Tubes

Dear Mr. Grier:

This is confirm my phone call on May 15, 1981 to report Reportable Occurrence 81-009 at Ginna Station. A program of planned Eddy Current examinations was conducted on both Loop "A" and "B" Steam Generators at Ginna Station from May 7th to 12th, 1981 during the present refueling and maintenance outage. This examination consisted of a multi-frequency Eddy Current examination to detect and measure potential tube defects. The inlets of both steam generators received a 100% inspection and the outlets received approximately a 25% inspection.

The results of these examinations did not reveal any reportable indications in the outlets of either steam generator and the inlet of the "A" steam generator. The "B" steam generator inlet has 45 tubes with indications from the multi-frequency data of which 44 are crevice indications in the tube sheet crevice area and 1 other indication above the tube sheet out on the periphery. Of these 44 tubes 6 tubes were above our 40% degradation criteria.

Based upon these results steam generator tube sleeving shall commence by sleeving the 44 crevice indication tubes and plugging the 1 tube in the periphery. After tube sleeving operations are complete tube pulling will be initiated to remove 3 tubes from the "B" generator hot leg.

Very truly yours,

Bruce A. Snow

Bruce A. Snow
Plant Superintendent

BAS:lhj

xc: J. E. Maier
L. S. Lang
J. C. Noon

A002
S1/1

366

U. S. NUCLEAR REGULATORY COMMISSION

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02

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09 SYSTEM CODE (11) CAUSE CODE (12) CAUSE SUBCODE (13) COMPONENT CODE (14) COMP. SUBCODE (15) VALVE SUBCODE (16)

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ACTION TAKEN (18) FUTURE ACTION (19) EFFECT ON PLANT (20) SHUTDOWN METHOD (21) HOURS (22) ATTACHMENT SUBMITTED (23) NPRD-4 FORM SUB. (24) PRIME COMP. SUPPLIER (25) COMPONENT MANUFACTURER (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

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15 FACILITY STATUS (28) % POWER (29) OTHER STATUS (30) METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)

16 ACTIVITY RELEASED (33) CONTENT OF RELEASE (34) AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

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18 PERSONNEL INJURIES NUMBER (40) DESCRIPTION (41)

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