

ORDER FOR MODIFICATION OF LICENSE

(EVENT V)

GINNA PLANT

DOCKET NO. 50-244

Insert the following pages in the Technical Specifications.

Pages-

3.3-2

3.3-3 *

3.3-4

4.3-2

*Included for format purposes only.

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- a. The refueling water tank contains not less than 230,000 gallons of water, with a boron concentration of at least 2000 ppm.
- b. Each accumulator is pressurized to at least 700 psig with an indicator level of at least 50% and a maximum of 82% with a boron concentration of at least 1800 ppm. Neither accumulator may be isolated.
- c. Three safety injection pumps are operable.
- d. Two residual heat removal pumps are operable.
- e. Two residual heat exchangers are operable.
- f. All valves, interlocks and piping associated with the above components which are required to function during accident conditions are operable.
- g. A.C. Power shall be removed from the following valves with the valves in the open position: safety injection cold leg injection valves 878B and D, accumulator injection valves 841 and 865, and refueling water storage tank delivery valves 856. A.C. power shall be removed from safety injection hot leg injection valves 878A and C with the valves closed. As soon as appropriate modifications are complete, D.C. control power shall be removed from refueling water storage tank delivery valves 896A and B with the valves open. In the meantime, single failure protection for valves 896A and B will be provided by locking out A.C. power, remote from the control room, with operating personnel assigned specifically to restore A.C. power when the valves are required to function in the event of a loss-of-coolant accident.
- h. Revisions to procedures for post-LOCA long term cooling as described in letters to the Nuclear Regulatory Commission from Rochester Gas and Electric Corporation dated April 1, 1975, April 30, 1975, and May 13, 1975, shall be implemented prior to reactor startup following the shutdown of March 10, 1975.
- i. Check valves 853A, 853B, 867A, 867B, 878G, and 878J shall be operable with less than 5.0 gpm leakage each. The leakage requirements of Technical Specification 3.1.5.1 are still applicable.

3.3-2

NRC Order dated
April 20, 1981

Amendment No. 7,

3.3.1.2 During power operation, the requirements of 3.3.1.1 may be modified to allow one of the following components to be inoperable at any one time. If the system is not restored to meet the requirements of 3.3.1.1 within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 3.3.1.1 are not satisfied within an additional 48 hours the reactor shall be placed in the cold shutdown condition.

- a. One accumulator may be isolated for a period of up to one hour.
- b. One safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours. The other two safety injection pumps shall be tested to demonstrate operability prior to initiating repair of the inoperable pump.
- c. One residual heat removal pump may be out of service provided the pump is restored to operable status within 24 hours. In addition, the following conditions must be satisfied.
 - (i) The other residual heat removal pump shall be tested to demonstrate operability prior to initiating repair of the inoperable pump.



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(ii) The two reactor coolant drain tank pumps shall be tested and their operability demonstrated prior to initiating repairs of the inoperable residual heat removal pump.

- d. One residual heat exchanger may be out of service for a period of no more than 24 hours.
- e. Any valve required for the functioning of the safety injection or residual heat removal systems may be inoperable provided repairs are completed within 12 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be tested to demonstrate operability.
- f. Power may be restored to any valve referenced in 3.3.1.1 g for the purposes of valve testing providing no more than one such valve has power restored and provided testing is completed and power removed within 12 hours.
- g. Those check valves specified in 3.3.1.1 i may be inoperable (greater than 5.0 gpm leakage) provided the inline MOVs are de-energised closed and repairs are completed within 12 hours.

3.3.1.3 Except during diesel generator load and safeguard sequence testing or when the vessel head is removed or the steam generator manway is open no more than one safety injection pump shall be operable whenever the temperature of one or more of the RCS cold legs is $\leq 330^{\circ}\text{F}$.

3.3.1.3.1 Whenever only one safety injection pump may be operable by 3.3.1.3 at least two of the three safety injection pumps shall be demonstrated inoperable a minimum of once per twelve hours by verifying that the control switches are in the pull-stop position.

3.3.2 Containment Cooling and Iodine Removal

3.3.2.1 The reactor shall not be made critical except for low temperature physics tests, unless the following conditions are met:

- a. The spray additive tank contains not less than 4500 gallons of solution with a sodium hydroxide concentration of not less than 30% by weight.
- b. At least two containment spray pumps are operable.
- c. Four fan cooler units are operable.



4.3.3 Check Valves

- 4.3.3.1 Leakage testing of check valves 853A, 853B, 867A, 867B, 878G and 878J shall be accomplished prior to criticality following (1) refueling, (2) cold shutdown, and (3) maintenance, repair or replacement work on the valves. Leakage may be measured indirectly from the performance of pressure indicators, system volume measurements or by direct measurement. Minimum test differential pressure shall be greater than 150 psid. See 4.3.3.4 for allowable leakage rates.
- 4.3.3.2 Check valves 878G and 878J will be tested for leakage following each safety injection flow test. Minimum test differential pressure shall be greater than 150 psid. See 4.3.3.4 for allowable leakage rates.
- 4.3.3.3 Motor-operated valves 878A and 878C and check valves 877A, 877B, 878F, and 878H shall be tested at the first refueling outage following the date of this order* to individually assure integrity in each hot leg high-head safety injection path. Testing shall also be performed after any opening of either motor-operated valve and at a minimum, once every 40 months. Opening of the motor-operated valves, and testing, are to be performed at a test pressure less than that of the lowest design pressure of any portion of the high-head safety injection system which may be pressurized during the test. Minimum test differential pressure shall be greater than 150 psid. See 4.3.3.4 for allowable leakage rates.
- 4.3.3.4 Allowable check valve leakage rates are as follows:
- (a) Leakage rates less than or equal to 1.0 gpm are considered acceptable.
 - (b) Leakage rates greater than 1.0 gpm but less than or equal to 5.0 gpm are considered acceptable if the latest measured rate has not exceeded the rate determined by the previous test by an amount that reduces the margin between measured leakage rate and the maximum permissible rate of 5.0 gpm by 50% or greater.
 - (c) Leakage rates greater than 1.0 gpm but less than or equal to 5.0 gpm are considered unacceptable if the latest measured rate exceeded the rate determined by the previous test by an amount that reduces the margin between measured leakage rate and the maximum permissible rate of 5.0 gpm by 50% or greater.
 - (d) Leakage rates greater than 5.0 gpm are considered unacceptable.

Basis: This material surveillance program monitors changes in the fracture toughness properties of ferritic materials in the reactor vessel beltline region of the reactor resulting from exposure to neutron irradiation and the thermal environment. The test data obtained from this program will be used to determine the conditions under which the reactor vessel can be operated with adequate margins of safety against fracture throughout its service life.

