APR 4 1981

MEMORANDUM FOR: Thomas H. Novak, Assistant Director
for Operating Reactors
Division of Licensing

FROM: V. S. Noonan, Assistant Director
for Materials & Qualifications Engineering
Division of Engineering

SUBJECT: GINNA STATION STEAM GENERATOR TUBES INSERVICE INSPECTION PROGRAM FOR THE 1980-1989 INTERVAL (TAC #43196)

Plant Name: R. E. Ginna Unit No. 1
Suppliers: Westinghouse; Gilbert Associates
Docket Number: 50-244
Responsible Branch and Project Manager: ORB#5; R. P. Snaider
Reviewer: D. T. Huang
Description of Task: Review of Ginna Station's Steam Generator Tubes Inservice Inspection Program for the 1980-1989 Interval
Review Status: Additional Information Needed

The Inservice Inspection Section of the Materials Engineering Branch, Division of Engineering has reviewed that portion of Rochester Gas and Electric Corporation's submittal dated November 6, 1980 regarding the Ginna Station Steam Generator Tubes Inservice Inspection Program for the 1980-1989 interval. We conclude in the basis of our review that the Ginna Station Steam Generator Tube Inservice Inspection Program for the 1980-1989 Interval is acceptable if the following two conditions are met:

1) Plugging limit for the ten test sleeves be established.

2) Two typographical errors mentioned in our Safety Evaluation be corrected.

Our Safety Evaluation is attached.

/S/

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Vincent S. Noonan, Assistant Director
Materials & Qualifications Engineering
Division of Engineering

Contact: D. T. Huang
X27377

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R. E. GINNA NUCLEAR POWER PLANT
REVIEW OF THE STEAM GENERATOR TUBE INSERVICE INSPECTION PROGRAM
FOR THE 1980 TO 1989 INTERVAL
SAFETY EVALUATION REPORT

MATERIALS ENGINEERING BRANCH
INSERVICE INSPECTION SECTION

INTRODUCTION

By letter dated November 6, 1980, Rochester Gas and Electric Corporation (the licensee) submitted the "Ginna Station Inservice Inspection Program for the 1980-1989 Interval" for review. Changes have been incorporated into Paragraph 5.7.1.1 and 6.5 of the Ginna Inservice Inspection Program in order to permit the installation of a maximum of ten test sleeves in steam generator tubes which would otherwise require plugging. These changes permit sleeving instead of plugging of tubes with unacceptable defects.

EVALUATION

We have reviewed the licensee's submittal dated November 6, 1980 regarding the Ginna Inservice Inspection Program. Based upon our review, we conclude that this inspection program meets the recommendations of Regulatory Guide 1.83, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes," Revision 1 and the requirements of Section XI of ASME Code with respect to the inspection methods to be used, provisions for a baseline inspections, selection and sampling of tubes, and inspection intervals. However, the Ginna Inservice Inspection Program is incomplete with respect to the installation of ten test sleeves in steam generator tubes, since it does not contain actions to be taken in the event defects are identified in the test sleeves, e.g. plugging limit for the ten test sleeves. Furthermore, two typographical errors should be revised to convey the same meaning regarding scope of

steam generators inservice inspection as originally intended. They are as follows:

1. The reference number in Paragraph 3.5 should be changed to 5 in order to be consistent with the reference list.
2. Paragraph 5.7.1.1 should be revised to indicate that the plant may resume operation only when both conditions (a) and (b) are met.

In conclusion, we find that the steam generator tube inservice inspection portion of the "Ginna Station Inservice Inspection Program for the 1980-1989 Interval" is acceptable with the condition that plugging limit for the ten test sleeves be established and the two typographical errors mentioned above be corrected.