

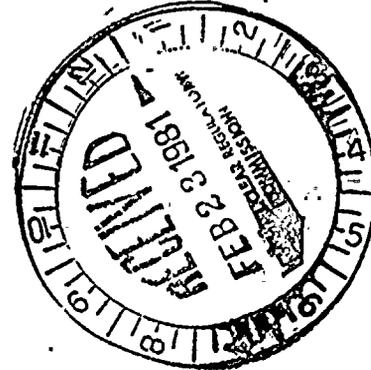


UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 17, 1980

Docket No. 50-244  
LS05-80-056

Mr. John Maier  
Vice President  
Electric and Steam Production  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, New York 14649



Dear Mr. Maier:

SUBJECT: FIRE PROTECTION - GINNA

The Commission has issued the enclosed Supplement No. 1 to the February 14, 1979 Fire Protection Safety Evaluation (FPSE) for the Robert E. Ginna Nuclear Power Plant. The FPSE had identified a number of items as being incomplete, thus requiring information from Rochester Gas and Electric Corporation and review by the NRC staff. You provided additional information in a submittal dated June 19, 1980. Based on our review of this information we have determined that your proposed modifications regarding eleven items are acceptable with regard to fire protection. These items are:

- 3.1.2 Water Suppression System
- 3.1.11 Battery Room Ventilation
- 3.1.21 Control Room Separation
- 3.1.24 Piping and Duct Penetrations
- 3.1.25 Construction Joints
- 3.1.39 RC Pump Lube Oil Collection System
- 3.1.48 Hydrogen Piping
- 3.2.2 Cable Separation
- 3.2.5 Electrical Cable Penetrations
- 3.2.6 Backflow Protection
- 3.2.7 Fire Pump Performance

Our Safety Evaluation for these items is attached as Enclosure 1 to this letter.

The required completion dates for the modifications associated with the items accepted in this supplement are specified by paragraph (d) of 10 CFR 50.48 using the date of this supplement as the date of the NRC staff Fire Protection Safety Evaluation Report accepting or requiring such features.

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Item 3.2.1, Shutdown Analysis, remains an open item and will be evaluated at a later date. You had submitted a Safe Shutdown Report dated December 28, 1979. By letter dated September 18, 1980, we informed you of our conclusion that the modifications required at Ginna to provide alternate shutdown capability are extensive and have not been fully identified and described. In addition, Enclosure 2 to our letter of November 24, 1980 (which forwarded the new Appendix R to 10 CFR 50 - Fire Protection Program for Operating Nuclear Power Plants) stated that you should install a dedicated shutdown system meeting the requirements of Section III Paragraph L of Appendix R.

As you are aware, the revised section 10 CFR 50-48 and Appendix R become effective February 17, 1981. Paragraph 50.48 (c)(5) requires that you submit plans and schedules for the installation of the dedicated shutdown system within 30 days of the effective date of the rule (i.e., by March 19, 1981). Further, Paragraph 50.48 (d)(4) will require that the dedicated shutdown system be installed within 30 months of NRC approval.

Sincerely,

*15*

Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

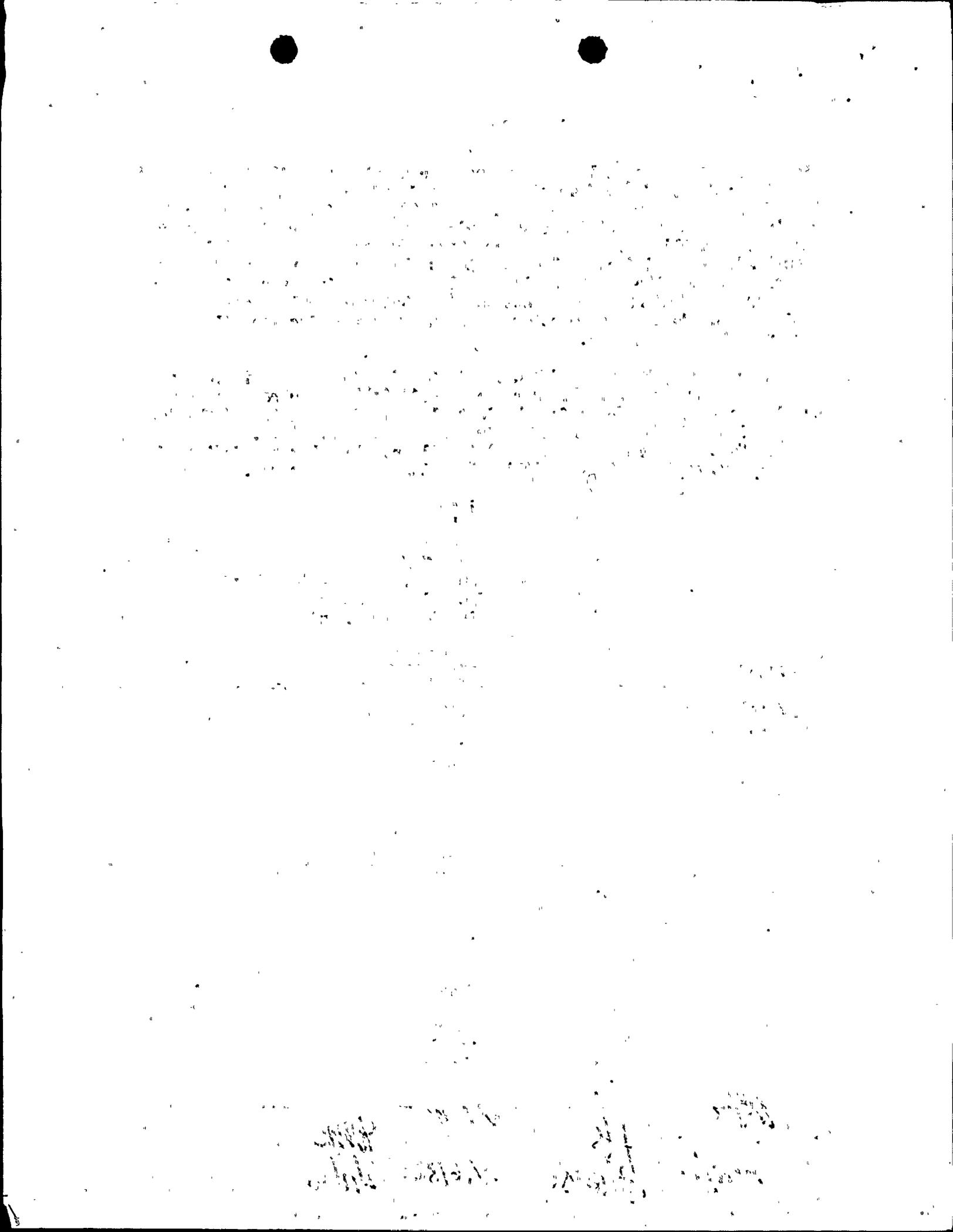
Enclosures:  
As stated

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December 17, 1980

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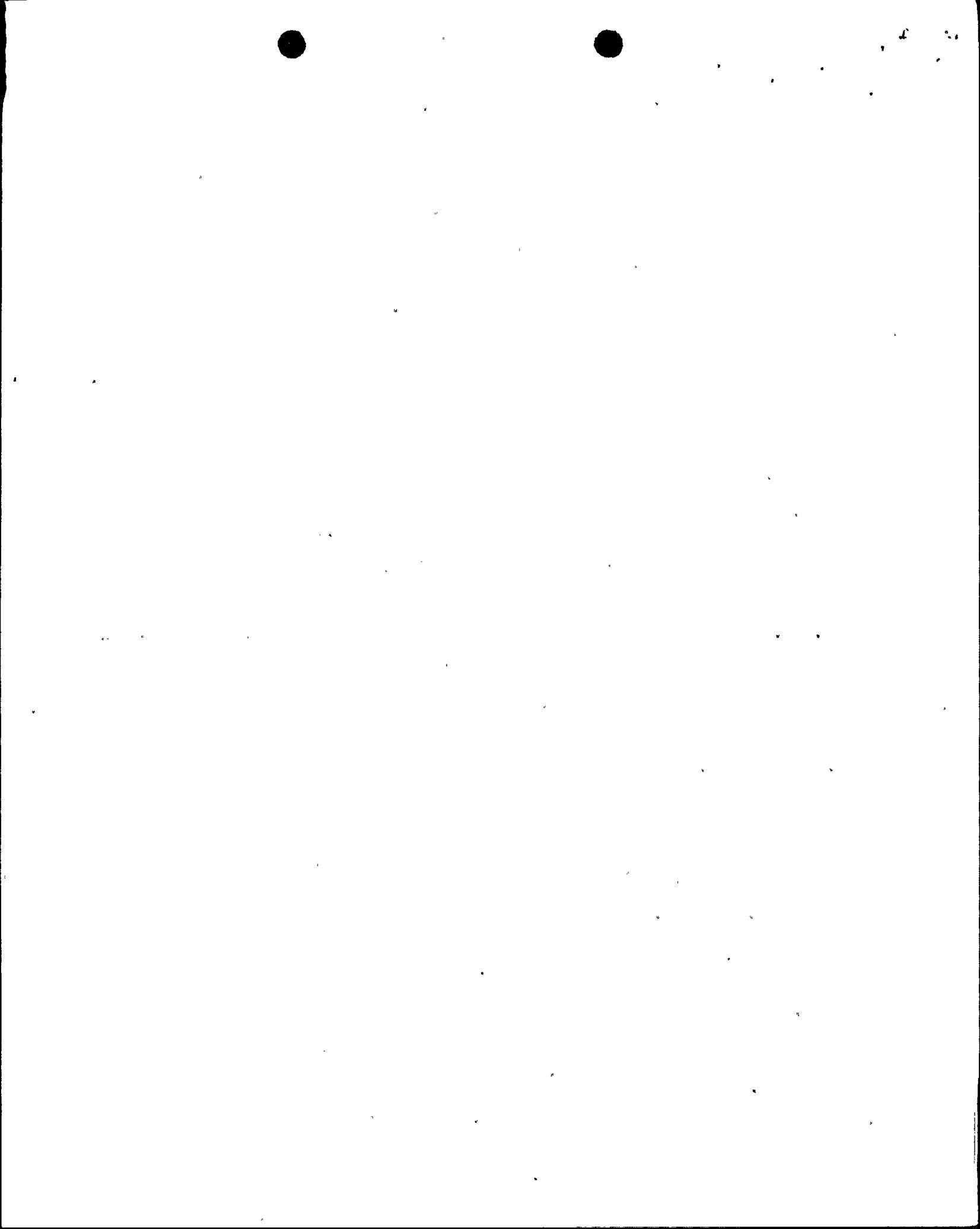
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Sincerely,

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing.

Enclosures:  
As stated

cc w/enclosures:  
See next page



Mr. Leon D. White, Jr.

- 3 -

December 17, 1980

cc

Harry H. Voigt, Esquire  
LeBoeuf, Lamb, Leiby and MacRae  
1333 New Hampshire Avenue, N. W.  
Suite 1100  
Washington, D. C. 20036

Mr. Michael Slade  
12 Trailwood Circle  
Rochester, New York 14618

Rochester Committee for  
Scientific Information  
Robert E. Lee, Ph.D.  
P. O. Box 5236 River Campus  
Station  
Rochester, New York 14627

Jeffrey Cohen  
New York State Energy Office  
Swan Street Building  
Core 1, Second Floor  
Empire State Plaza  
Albany, New York 12223

Director, Technical Development  
Programs  
State of New York Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

Rochester Public Library  
115 South Avenue  
Rochester, New York 14604

Supervisor of the Town  
of Ontario  
107 Ridge Road West  
Ontario, New York 14519

Resident Inspector  
R. E. Ginna Plant  
c/o U. S. NRC  
1503 Lake Road  
Ontario, New York 14519

Ezra I. Bialik  
Assistant Attorney General  
Environmental Protection Bureau  
New York State Department of Law  
2 World Trade Center  
New York, New York 10047

Director, Criteria and Standards  
Division  
Office of Radiation Programs  
(ANR-460)  
U. S. Environmental Protection  
Agency  
Washington, D. C. 20460

U. S. Environmental Protection  
Agency  
Region II Office  
ATTN: EIS COORDINATOR  
26 Federal Plaza  
New York, New York 10007

Herbert Grossman, Esq., Chairman  
Atomic Safety and Licensing Board  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dr. Richard F. Cole  
Atomic Safety and Licensing Board  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dr. Emmeth A. Luebke  
Atomic Safety and Licensing Board  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Mr. Thomas B. Cochran  
Natural Resources Defense Council, Inc.  
1725 I Street, N. W.  
Suite 600  
Washington, D. C. 20006