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AUTH.NAME AUTHOR AFFILIATION
WHITE,L.D. Rochester Gas & Electric Corp.
RECIP.NAME RECIPIENT AFFILIATION
CRUTCHFIELD,D. Operating Reactors Branch 5

SUBJECT: Forwards responses to SEP Topics III-6 re seismic design considerations & III-11 re component integrity. Detailed schedule for future analyses required for completion open items will be provided as soon as possible.

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LEON D. WHITE, JR.
Executive Vice President

TELEPHONE
AREA CODE 716 546-2700

February 6, 1981

Director of Nuclear Reactor Regulation
ATTN: Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: SEP Topics III-6 and III-11, "Seismic Design Considerations"
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Crutchfield:

We have reviewed your letter of January 7, 1981 (received January 15, 1981) and the attached NUREG/CR-1821, "Seismic Review of the Robert E. Ginna Nuclear Power Plant as Part of the Systematic Evaluation Program". The attachment and enclosed reports respond to most of the NRC concerns. The very short time frame allowed for response does not enable us to submit a detailed schedule of future analyses required for the completion of some items. As noted in the attachment, a schedule for completion of future analyses will be provided as soon as it can be reasonably generated.

Very truly yours,

L. D. White, Jr.

Enclosures

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Attachment

Open Issues, R.E. Ginna Seismic Review

1,2. Auxiliary Building and Turbine Building Bracing. Rochester Gas and Electric is in the process of developing a Structural Review Evaluation in response to the concerns expressed in this report. Because of the short time between receipt of NUREG/CR-1821 (January 15, 1981) and the required response date (February 6, 1981), a detailed schedule is not yet available. The schedule for evaluation of this bracing will be provided by March 23, 1981.

3. Essential Service Water Pumps. As noted in the recommendation, RG&E is evaluating the feasibility of making the modifications required to provide additional seismic margin for the Service Water Pumps, above that provided for in the original design methodology. Concurrently, RG&E is studying the requirements for a "dedicated shutdown system", which could include a provision for Service Water. The RG&E response concerning the dedicated shutdown system will be submitted by March 19, 1981.

It is important to note that no immediate steps to correct the Service Water System are warranted. A means of safe shutdown without Service Water has been reviewed by the NRC staff and deemed acceptable as an interim means. This shutdown method was described in RG&E's "Fire Protection-Shutdown Analysis" dated December 28, 1979. It was noted as an acceptable interim measure in the staff's "SEP Review of Pipe Break Outside Containment, Topic III-5.B", transmitted by letter of June 14, 1980 from Dennis M. Crutchfield to Leon D. White, Jr.

4,5. Component Cooling Surge Tank and RWST. The design of measures to increase the seismic margins above those provided by the original design methodology is presently under study. Because of the short time available between receipt of NUREG/CR-1821 (January 15, 1981) and the required response date (February 6, 1981) a detailed schedule for analysis and modification has not yet been generated. A schedule for performing this evaluation will be provided by March 23, 1981. It is expected that the results of the evaluation can be made available on or about June 30, 1981.

6. Motor-Operated Valves. As recommended in NUREG/CR-1821, the Ginna Piping Seismic Upgrade Program will account for the stresses induced by valve eccentricity into the piping analysis. The analysis will be performed in

accordance with the "Design Criteria, Ginna Station, Seismic Upgrade Program", Revision 1, dated September 1, 1980.

7-10. Steam Generator, Reactor Coolant Pump, Pressurizer, and Control Rod Drive Mechanism. The following stress reports are included for review:

a) Vertical Steam Generator Stress Report", Wedler, R.P., April 1969, Westinghouse, Tampa.

b) "44 Series Steam Generator External Load Analysis Update", Nagayasu, K., De Rosa, P. P., May 1975, Westinghouse, Tampa.

c) "L-106 High Speed Control Rod Drive Mechanism Stress Analysis", Ganguly, S.K., June 1968, Westinghouse, Cheswick.

d) "RGE Seismic Frame Calculations", Howell, R., Westinghouse Commercial Atomic Power Division, Monroeville.

e) "Seismic Analysis of Shaft Seal Reactor Coolant Pump", van Allen, F.K., July 1968, Westinghouse, Cheswick.

f) "Pressurizer Design Report," Mendolia, F.J. and DeRosa, P.P., September 1969, Westinghouse, Tampa.

11. Battery Racks. The modification to replace the wooden battery racks are scheduled to be completed during the Spring 1981 refueling outage.

12.13. Motor Control Centers and Switchgear. These items are being evaluated in conjunction with the "Seismic Operability" program being performed as an SEP Owners Group project. The Owners Group has hired Westinghouse to lead this program. A meeting to present the proposed program is presently scheduled for March 5, 1981.

14. Control Room Electrical Panels. Proper anchor bolt design and placement was verified in conjunction with the "Anchorage and Support of Safety-Related Electrical Equipment" program, first described to the NRC in a letter from RG&E dated February 11, 1980. The status of this program was transmitted to the NRC by letter of December 22, 1980.

The evaluation of internally-mounted equipment is being performed in the "Seismic Operability" program, noted in 12, 13, above.

15. Electrical Cable Raceways. The seismic qualification program for cable trays is being performed as an SEP Owners Group project. This program was discussed in a letter from Richard E. Schaffstall (KMC) to

Dennis M. Crutchfield (NRC) in a letter dated August 28, 1980.

Since that time, the SEP Owners Group has hired URS/Blume to lead this program. URS/Blume made a presentation to the NRC staff concerning this program on October 3, 1980. A meeting to update the staff on the progress of the program is presently scheduled for March 5, 1981.