

REGULATORY INFORMATION DISTRIBUTION SYSTEM (GRIDS)

ACCESSION NBR: 8202240083 DOC. DATE: 82/02/18 NOTARIZED: NO DOCKET #
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 MAIER, J. E. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 CRUTCHFIELD, D. Operating Reactors Branch 5

SUBJECT: Informs that schedule for performing analyses of code changes that could have decreased margins of safety will be submitted by 820531, per NRC 811230 ltr forwarding draft SER. Containment vessel evaluation encl.

DISTRIBUTION CODE: A0355 COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2+34
 TITLE: SEP Topics

NOTES: 1 copy: SEP Sect. Ldr.

05000244

ACTION:	RECIPIENT ID CODE/NAME		COPIES		RECIPIENT ID CODE/NAME		COPIES	
			LTTR	ENCL			LTTR	ENCL
	ORB #5 BC	01	7	7				
INTERNAL:	IE	06	2	2	NRR/DE/ADMGE	13	1	1
	NRR/DE/HGEB	10	2	2	NRR/DL/ORAB	11	1	1
	NRR/DL/SEP8	12	3	3	NRR/DSI/AEB		1	1
	NRR/DSI/CSB	07	1	1	REG FILE	04	1	1
EXTERNAL:	ACRS	14	10	10	LPDR	03	1	1
	NRC PDR	02	1	1	NTIS	5	1	1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 33 ENCL 33
~~32~~ ~~32~~

JW

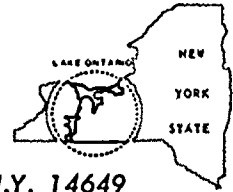


1. The first part of the document discusses the importance of maintaining accurate records of all transactions. It emphasizes that this is crucial for ensuring the integrity of the financial statements and for providing a clear audit trail.

2. The second part of the document outlines the specific procedures that should be followed when recording transactions. This includes details on how to handle receipts, invoices, and other supporting documents, as well as the timing and frequency of record-keeping.

3. The third part of the document addresses the role of internal controls in the record-keeping process. It discusses how these controls can help to prevent errors and fraud, and how they should be designed and implemented to ensure the reliability of the data.

4. The fourth part of the document provides a summary of the key points discussed and offers some final thoughts on the importance of a robust record-keeping system. It concludes by stating that a well-maintained system is essential for the success of any business or organization.



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

JOHN E. MAIER
Vice President

TELEPHONE
AREA CODE 716 546-2700

February 18, 1982

Director of Nuclear Reactor Regulation
Attention: Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Subject: SEP Topic III-7.B, "Design Codes, Design Criteria
and Loading Combinations"
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Crutchfield:

This letter is in response to your letter of December 30, 1981, transmitting the draft Safety Evaluation Report for this topic, as well as two contractor reports which formed the bases for the SER.

The first contractor report, Franklin Research Center draft Technical Evaluation Report TER-C5257-322, "Design Codes, Design Criteria, and Loading Combinations," identifies code changes that have occurred that could have decreased margins of safety. RG&E was requested to assess the safety margins where such code changes have been identified. This is a substantial task, requiring the review, comparison, and analysis of major sections of the ASME, AISC, and ACI Codes. We have not yet been able to assess the full scope of effort needed to perform this evaluation. It is expected that a schedule for performing the necessary analyses will be submitted to the NRC by the end of May, 1982.

The second contractor report, Structural Mechanics Associates Report SMA 12205.27R, "Combined Loads Evaluation," dated December 1981, addresses a potential containment liner plate integrity problem. The enclosed Gilbert/Commonwealth report, "Containment Vessel Evaluation," responds to the SMA report. The conclusion of the G/C report is that the structural integrity of the studs and liner would be maintained.

A035
5/11

8202240083 820218
PDR ADOCK 05000244
PDR



[The text in this section is extremely faint and illegible, appearing as scattered black specks and light gray smudges across the page.]

DATE February 18, 1982
TO Mr. Dennis M. Crutchfield

2

It should be noted that the specific G/C analysis was performed for a post-accident pressure-temperature profile which was calculated by G/C (see Figure 3 of the report). This corresponds fairly closely to the Ginna FSAR curves and the August 1981 SMA preliminary draft report. (A February 1, 1982 RG&E letter relative to SEP Topics VI-2.D and VI-3 notes that the steam line break conditions used as the basis for the December 1981 SMA report are excessively conservative to use as a design basis). Further, the G/C report primarily evaluates this August 1981 SMA report, rather than the December 1981 report. However, the conclusion of the G/C report, provided in Sections V and VI, is that the containment liner and stud integrity would be maintained, even using these overly-conservative NRC steam break conditions.

Very truly yours,


John E. Maier

