



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

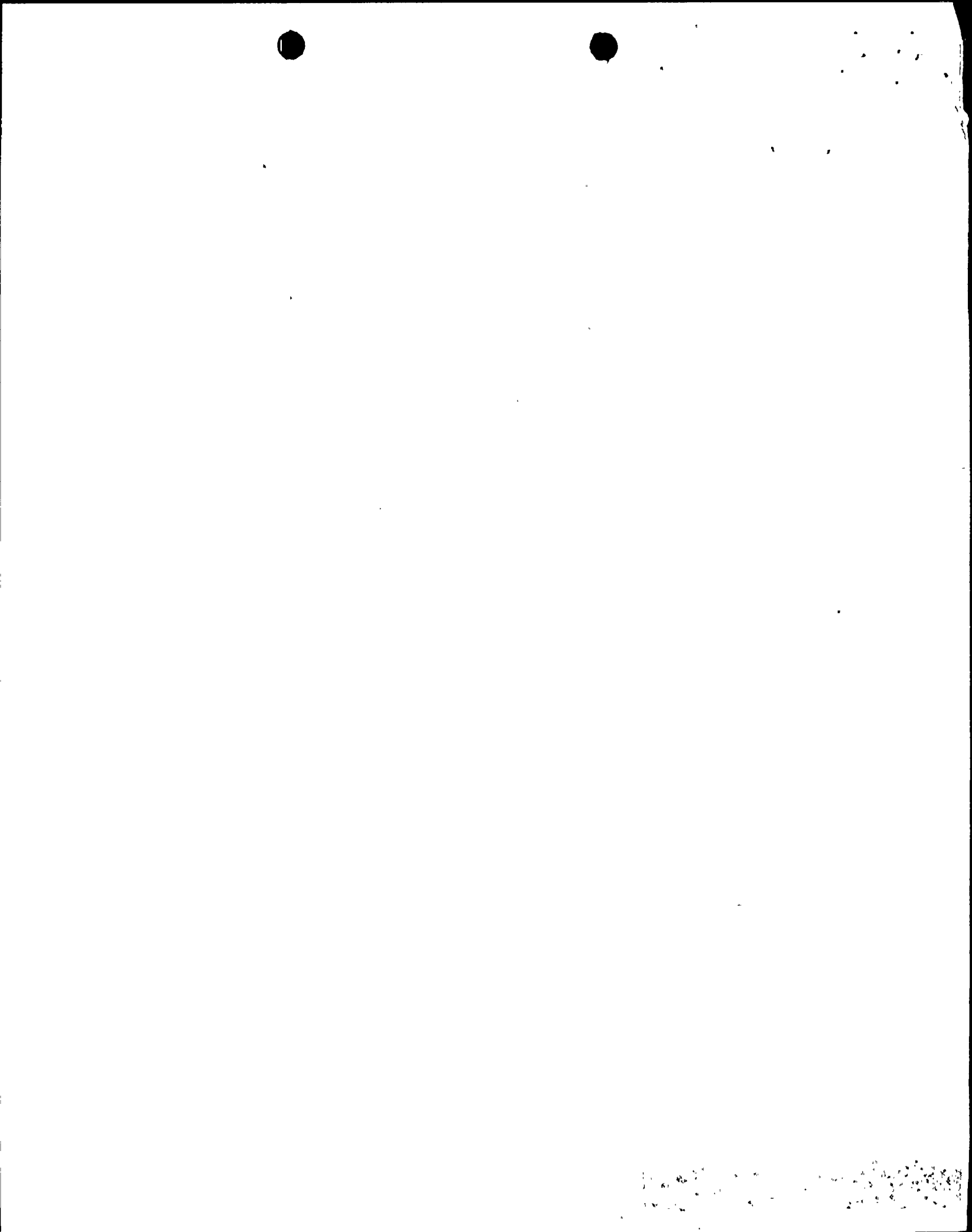
DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 48
License No. DPR-18

- 1: The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) notarized January 14, 1982 (transmitted by letter dated January 14, 1982), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.




2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-18 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 48, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective on January 13, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 3, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 48

PROVISIONAL OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines which indicate the area of changes.

PAGES

3.3-3

3.3-4

3.3.1.2 During power operation, the requirements of 3.3.1.1 may be modified to allow components to be inoperable at any one time. More than one component may be inoperable at any one time provided that one train of the ECCS is operable. If the requirements of 3.3.1.1 are not satisfied within the time period specified below, the reactor shall be placed in hot shutdown within 6 hours and at $T_{avg} < 350^{\circ}\text{F}$ in an additional 6 hours.

- a. One accumulator may be inoperable or isolated for a period of up to one hour.
- b. One safety injection pump may be out of service provided the pump is restored to operable status within 72 hours. The other two safety injection pumps shall be tested to demonstrate operability prior to initiating repair of the inoperable pump.
- c. One residual heat removal pump may be out of service provided the pump is restored to operable status within 72 hours. The other residual heat removal pump shall be tested to demonstrate operability prior to initiating repair of the inoperable pump.

- d. One residual heat exchanger may be out of service for a period of no more than 72 hours.
- e. Any valve, interlock, or piping required for the functioning of one safety injection train and/or one low head safety injection train (RHR) may be inoperable provided repairs are completed within 72 hours. Prior to initiating valve repairs, all valves in the system that provide the duplicate function shall be tested to demonstrate operability.
- f. Power may be restored to any valve referenced in 3.3.1.1 g for the purposes of valve testing providing no more than one such valve has power restored and provided testing is completed and power removed within 12 hours.
- g. Those check valves specified in 3.3.1.1 i may be inoperable (greater than 5.0 gpm leakage) provided the inline MOVs are de-energized closed and repairs are completed within 12 hours.

3.3.1.3 Except during diesel generator load and safeguard sequence testing or when the vessel head is removed or the steam generator manway is open, no more than one safety injection pump shall be operable whenever the temperature of one or more of the RCS cold legs is $\leq 330^{\circ}\text{F}$.

3.3.1.3.1 Whenever only one safety injection pump may be operable by 3.3.1.3, at least two of the three safety injection pumps shall be demonstrated inoperable a minimum of once per twelve hours by verifying that the control switches are in the pull-stop position.

3.3.2 Containment Cooling and Iodine Removal

3.3.2.1 The reactor shall not be made critical except for low temperature physics tests, unless the following conditions are met:

- a. The spray additive tank contains not less than 4500 gallons of solution with a sodium hydroxide concentration of not less than 30% by weight.
- b. At least two containment spray pumps are operable.
- c. Four fan cooler units are operable.



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 48 TO PROVISIONAL OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By application notarized January 14, 1982 (submitted by letter dated January 14, 1982), Rochester Gas and Electric Corporation (RG&E) requested changes to the Technical Specifications for the R. E. Ginna Nuclear Power Plant. These changes would revise the specifications dealing with the availability of components in the Safety Injection and Residual Heat Removal (RHR) Systems.

2.0 BACKGROUND

During routine surveillance, two pinhole leaks were discovered in the RHR pump suction piping from one of the containment sumps. In order to repair the system the affected RHR pump and one of the two Reactor Coolant Drain Tank (RCDT) pumps had to be isolated. This would be in violation of the technical specifications that require testing the other RHR pump and both of the RCDT pumps to demonstrate their operability. Emergency relief from this technical specification requirement was granted by our telephone authorization given January 13, 1982 (confirmed by our letter dated January 15, 1982), as requested by RG&E's letter telecopied to us on January 13, 1982. The application notarized January 14, 1982 (submitted by letter dated January 14, 1982), is the formal request for this technical specification change.

3.0 EVALUATION

The proposed change would clarify existing Technical Specifications. Specification 3.3.1.2, which addresses requirements in the event of ECCS subsystem inoperability, would be revised to eliminate requirements not based on safety analyses and to address all portions of the ECCS including piping.

The requirements regarding number of inoperable components is restated to ensure that one train is always operable. This restatement is necessary as a result of implementing the definition of "operability" as described

in RG&E's letter dated May 5, 1981. The time period for corrective action is restated and is consistent with accepted licensing practices. The specified time period remains short such that the likelihood of requiring remaining operable equipment is small. The requirement for RCDT pump operability in the event of a RHR pump inoperability has been deleted. The RCDT pumps are not required to satisfy any safety analysis assumptions. While they could provide added redundancy, they are substantially smaller than the RHR pumps so they cannot provide a comparable level of decay heat removal. Inoperability of piping and interlocks is added to the specification for completeness and the specifications are clarified to address only that portion of the RHR system required for the low head safety injection, including the recirculation phase.

4.0 SUMMARY

Based on our evaluation of the information provided by the licensee, we find that the proposed changes do not substantially change the existing requirements, but do provide for clarity and completeness. Therefore, we conclude that the proposed changes to the Technical Specifications are acceptable.

5.0 ENVIRONMENTAL CONSIDERATION

We have determined that the proposed amendment does not authorize a change in effluent types, increase in total amounts of effluents, or an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

We also conclude, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

Date: February 3, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-244ROCHESTER GAS AND ELECTRIC CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE.

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 48 to Provisional Operating License No. DPR-18, issued to Rochester Gas & Electric Corporation (the licensee), which revised the Technical Specifications for operation of the R. E. Ginna Plant (facility) located in Wayne County, New York. This amendment became effective on January 13, 1982.

The amendment incorporates technical specification changes that clarify the requirements for operability of the Emergency Core Cooling System.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.


The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.



For further details with respect to this action, see (1) the application for amendment notarized January 14, 1982 (transmitted by letter dated January 14, 1982), (2) Amendment No. 48 to License No. DPR-18, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this third day of February, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

