UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 49
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) notarized April 23, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

B. The facility will operate in conformity with the application, the provisins of the Act, and the rules and regulations of the Commission;

C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;

D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technial Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-18 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 49 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 23, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 49

PROVISIONAL OPERATING LICENSE NO. DPR-18

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Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines which indicate the area of changes.

PAGES
3.14-7
3.14-8

TABLE 3.14-1

FIRE DETECTION INSTRUMENTS.

INSTRUMENT LOCATION	MINIMUM INSTRUMENTS OPERABLE ***	
	HEAT	SMOKE
1. Containment		
"A" Post-Accident Charcoal Bank	3*	N/A
"B" Post-Accident Charcoal Bank	3*	N/A
"A" Aux. Filter Charcoal Bank	1*	N/A
"B" Aux. Filter Charcoal Bank	1*	N/A
Cable Trays Basement Elev.	1**	N/A
Cable Trays Intermed. Elev.	2**	N/A
Cable Trays Operating Floor	1**	N/A
"A" RCP Intermediate Floor	1**	N/A
"B" RCP Intermediate Floor	1**	N/A
Area Detection Operating Floor	N/A	7
2. Control Room		
Area and Cabinet	N/A	19
Control Room/Turb. Bldg. Wall	4	N/A
3. Relay Room	3	16
4. Computer Room		
Under Floor	N/A	3
Ceiling	N/A	3
5. Battery Rooms		
"A" Battery Room	N/A	1
"B" Battery Room	N/A	1
6. Control Building		
Air Handling Room	N/A	3
7. Diesel Generator		
"A" Generator Room	2	N/A
"A" Generator Vault	N/A	1
"B" Generator Room	2	N/A
"B" Generator Vault	N/A	1
8. Intermediate Building		
Motor Driven Aux. Fd. Pump Area	N/A	9
Turb. Driven Aux. Fd. Pump & Res.	1	N/A
Cable Trays Basement North	N/A	14
"A" Purge Filter Elev. 315'-4"	N/A	1
"B" Purge Filter Elev. 315'-4".	N/A	1

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INSTRUMENT LOCATION	MINIMUM INSTRUMENTS OPERABLE	
	HEAT	SMOKE
9. Screen House		
Area Detection Serv. Water Pump and Bus Area	N/A	11
Cable Trays Basement	N/A	4
10. Standby Auxiliary Feedwater Bldg.	N/A	8
11. Cable Tunnel	10	8
12. Auxiliary Building		
General Area	N/A	8
Area Basement East	N/A	5
Area Basement West and RHR Pit	N/A	9
Cable Trays/SI Pumps Basement	N/A	5
Penetration Area Cable Trays Mezz.	N/A	2
Cable Trays, Elec. Cab. Mezz. Center	N/A	4
Cable Trays Mezz. East	N/A	4
Area Operating Floor	N/A	13

* Resistance Temperature Detectors (RTD) Only
** Line Type Detectors
*** The fire detection instruments located within the containment are not required to be operable during the performance of Type A containment leakage rate tests.

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 49 TO PROVISIONAL OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By application notarized April 23, 1982, Rochester Gas and Electric Corporation (RG&E) requested changes to the Technical Specifications appended to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. These changes would revise the specifications dealing with the fire detection system.

2.0 BACKGROUND

The fire detectors at Ginna have been demonstrated to be sensitive to dust and particles which are stirred up during maintenance activities. Pressurization and depressurization of the containment during integrated leak rate testing may cause dust and particles to give spurious fire alarms from the containment. To avoid such spurious alarms RG&E has requested authorization to make the containment fire detection instruments inoperable during the containment leak rate test.

3.0 EVALUATION

During integrated leak rate tests, no maintenance activities will be taking place. RG&E conducts a final inspection of the containment after maintenance activities have ceased, prior to commencing the integrated leak rate tests. Therefore the risk of fire caused by welding, buring, grinding or other personnel activities will not exist.

During the integrated leak rate test the plant is at cold shutdown with the reactor coolant pumps secured. Therefore, the risk of fire caused by oil leaks onto hot equipment is significantly reduced from the risk at power operation.

The only significant fire hazards in containment during the test are the operating containment fan coolers. The fan cooler bearing vibration and air-plenum temperature instruments in the control room provide an indication of fan cooler malfunction. In addition, monitoring the containment temperature during the test will provide an indication of fire.

4.0 SUMMARY

Based on our evaluation of the information provided by the licensee, we find that making the fire detection system inoperable during the integrated leak rate tests will not result in a significant increase in the risk of fire and is consistent with current licensing practices. Therefore, we conclude that the proposed change to the Technical Specifications is acceptable.

5.0 ENVIRONMENTAL CONSIDERATION

We have determined that the proposed amendment does not authorize a change in effluent types, increase in total amounts of effluents, or an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

We also conclude, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin; the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

7.0 ACKNOWLEDGMENTS

The following NRC personnel has contributed to this evaluation:

JLyons

Date: April 23, 1982