

Attachment A

1. Replace Technical Specification page 3.6-6.

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PDR ADCK 05000244
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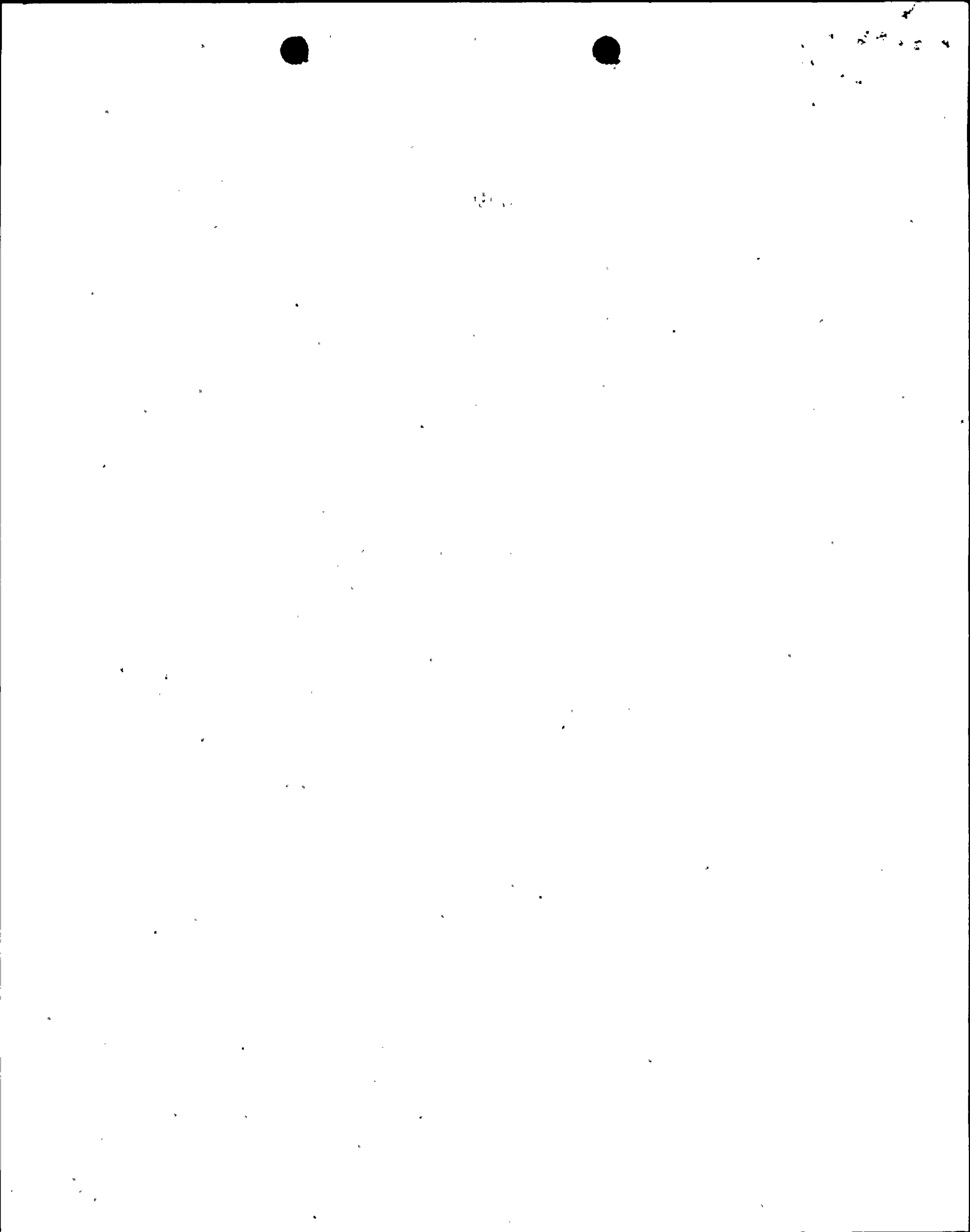


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PENT. NO.	IDENTIFICATION/DESCRIPTION	ISOLATION BOUNDARY	MAXIMUM ISOLATION TIME *(SEC)	ISOLATION BOUNDARY	MAXIMUM ISOLATION TIME *(SEC)
203	Contain. Press. transmitter PT-947 & 948	PT 947 PT 948	NA NA	MV 1819C MV 1819D	NA NA
203	Post accident air sample to "B" fan	MV 1563 MV 1566	NA NA	MV 1565 MV 1568	NA NA
204	Purge Supply Duct	AOV 5870	5	AOV 5869	5
205	Hot leg loop sample	AOV 966C	60	MV 956D(14)	NA
206	Przr. liquid space sample	AOV 966B	60	MV 956E(14)	NA
206	"A" S/G sample	AOV 5735	60	MV 5733(7)	NA
207	Przr. Steam space sample	AOV 966A	60	MV 956F	NA
207	"B" S/G sample	AOV 5736	60	MV 5734(7)	NA
209	Reactor Compart. cooling Units A & B	MV 4758(16) MV 4635(16)	NA NA	(11) (11)	NA NA
210	Oxygen makeup to A & B recombiners	MV 1080A	NA	SOV IV-2A SOV IV-2B	NA Normally Closed NA Normally Closed
300	Purge Exhaust Duct	AOV 5878	5	AOV 5879	5
301	Aux. steam supply to containment	MV 6151	NA	MV 6165(15)	NA
303	Aux. steam condensate return	MV 6175	NA	MV 6152(15)	NA
304	"A" Hydrogen recombiner (pilot and main)	MV 1084A MV 1076A	NA NA	SOV IV-5A SOV IV-3A	NA Normally Closed NA Normally Closed
305	Radiation Monitors R-11, R-12 & R-10A Auto Inlet Isol.	AOV 1597	60	MV 1596	NA
305	R-11, R-12 & R-10A Outlet	AOV 1599	60	AOV 1598	60
305	Post Accident air sample (containment)	MV 1554 MV 1557 MV 1560	NA NA NA	MV 1556 MV 1559 MV 1562	NA NA NA
307	Fire Service Water	CV 9229	NA	AOV 9227	(18)

3.6-6

Amendment No. 42, 50 Proposed



Attachment B
Safety Evaluation

The proposed Technical Specification change affects the containment isolation boundary for a single containment penetration, penetration 305, the containment radiation monitor return line. A chronic problem with the existing containment isolation check valve has necessitated a modification to this line. The existing check valve will be replaced by a new air-operated valve outside containment but close to the containment wall. The modification will improve the containment isolation valve reliability.

As a result of the modification, both automatic valves for isolation of this line will be outside containment. This type of arrangement has previously been evaluated to be acceptable in the Ginna SEP Integrated Plant Safety Assessment, NUREG-0821. The piping design pressure between the containment and the isolation valves is at least equal to the containment design pressure. The isolation valves are designed to function against containment pressure. The modified piping run between the containment and the new valve is as short as possible, approximately 3 feet, and is Seismic Category I. The piping supports for the new valve and piping between it and the containment wall are designed to subsection NF of the ASME code.

The NRC PRA study reported in NUREG-0821 found that the risk resulting from two valves outside containment is about the same as the risk resulting from one valve inside and one valve outside containment.

The modified piping configuration outside containment will not be susceptible to an event which could also cause a LOCA or event inside containment that would require containment isolation. Conversely, an event inside containment will not result in damage to the isolation piping outside containment.

The new valve to be installed will close on the same isolation signals as other valves in this radiation monitor system. Resetting of the containment isolation signal will not cause the valve to automatically reopen. The new valve will operate and reopen in conjunction with existing valve 1597, another valve in the same system and in a line within the same penetration, which allows passage of air from the containment to the radiation monitor. Electrical independence and single failure protection will be maintained. Ganged reopening of two valves within the same system and performing a single function meets the intent of NUREG-0737.

Based upon these reasons, the proposed modification is acceptable under General Design Criterion 56 on an "other defined basis" as permitted by the GDC.

Therefore, the piping modification and the Technical Specification change will not result in any undue risk to public and safety.

