UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of

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Rochester Gas and Electric Corporation (R. E. Ginna Nuclear Power Plant, Unit No. 1) Docket No. 50-244

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APPLICATION FOR AMENDMENT TO OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the U.S. Nuclear Regulatory Commission (the "Commission"), Rochester Gas and Electric Corporation ("RG&E"), holder of Provisional Operating License No. DPR-18, hereby requests that the Technical Specifications set forth in Appendix A to that license be amended. This change would revise the inservice inspection program ten year interval start dates and would incorporate a reference to the Pump and Valve Test Program.

The proposed technical specification changes are set forth in Attachment A to this Application. A safety evaluation is set forth in Attachment B. This evaluation also demonstrates that the proposed change does not involve a significant change in the types or a significant increase in the amounts of effluents or any change in the authorized power level of the facility. As demonstrated in Attachment C, the proposed change does not involve a significant hazards consideration. These changes involve a single safety issue and are in accordance with NRC position. Therefore, this change is a Class III Amendment in accordance with 10 CFR 170.22. •

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WHEREFORE, Applicant respectfully requests that Appendix A to Provisional Operating License No. DPR-18 be amended in the form attached hereto as Attachment A.

Rochester Gas and Electric Corporation

By John E. Maier Vice President,

Electric and Steam Production

Subscribed and sworn to before me , on this 19th day of December, 1983.

LYNN I. HAUCK NOTARY PUBLIC, State of N.Y., Monroe County My Commission Expires March 30, 1934 a and a High A ship a

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Attachment A

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1. Replace Technical Specification pages 4.2-1 and 4.2-2 with the enclosed pages.

4.2

Inservice Inspection

Applicability

Applies to the inservice inspection of Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator Tubes. It also applies to inservice pump and valve testing.

Objectives

To provide assurance of the continuing structural and operational integrity of the structures, components and systems in accordance with the requirements of 10 CFR 50.55a(g).

Specification

4.2.1

The inservice inspection program for Quality Groups A, B and C components, High Energy Piping Outside of Containment and Steam Generator tubes shall be in accordance with Appendix B of the Ginna Station Quality Assurance Manual. The inservice pump and valve testing program shall be in accordance with Appendix C of the Ginna Station Quality Assurance Manual. These inservice inspection programs shall define the specific requirements of the edition and Addenda of the ASME Boiler and Pressure Vessel Code, Section XI, which are applicable for the forty month period of the ten year inspection interval. The programs' ten year inspection intervals shall be based on the following commencing dates.

4.2.1.1

The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.

4.2.1.2

The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990 and 2000, respectively.

4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990

4.2-1

Amendment No. 18 Proposed

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and 2000, respectively. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

- 4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.
- 4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
- 4.2.1.6 The inspection interval for the Inservice Pump and Valve Testing Program shall be ten year intervals commencing with January 1, 1981, 1990 and 2000.

Basis:

The inservice inspection programs provide assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The programs comply with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures, components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirement of General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.

4.2-2

Amendment No. 1⁄3 Proposed

Attachment B

This change revises the second and subsequent ten year interval start dates of the Quality Group B and C programs and program for High Energy Piping outside of Containment to coincide with the interval of the Quality Group A program. The revised Group B and C interval start date is consistent with the revised Inservice Inspection Program for the 1980-1989 intervals submitted in July 1979. For consistency, consideration to change this High Energy Piping interval start date was subsequently considered. The interval start dates of the submitted Inservice Inspection Program are to be revised accordingly. The change also incorporates the Inservice Pump and Valve Testing Program and corresponding interval start dates for the second and subsequent intervals. This program, which originated as a supplement to the original program, is not currently referenced by Technical Specifications.

The revisions have increased the component surveillance during the 40 month transition period between January 1, 1980 and May 1, 1983 due to the interval 1 and 2 overlap. This change, therefore, will not affect the health and safety of the public.

Attachment C

In promulgating its interim final rule to comply with Public Law 97-415, the Commission published as guidelines a series of examples of amendments that are not likely to involve significant hazards considerations. 48 Fed. Reg. 14864, 14870. The proposed change revises the second and subsequent ten year interval start dates for the Quality Group B and C programs and for the High Energy Line Break outside containment program. The programs now begin January 1, 1980, January 1, 1990 and January 1, 2000. The proposed change also incorporates a reference to the Pump and Valve Test Program. Thus, the proposed revisions constitute additional limitations, that is, more restrictive requirements. The Commission's example (ii) clearly applies to this change. Therefore, the proposed change does not constitute a significant hazards consideration.

The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated; or create the possibility of a new or different kind of accident from any accident previously evaluated; or involve a significant reduction in a margin of safety.