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ACCESSION NBR: 8307260445 DOC. DATE: 83/07/20 NOTARIZED: NO DOCKET # 05000244
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G
 AUTH. NAME: MAIER, J. E. AUTHOR AFFILIATION: Rochester Gas & Electric Corp.
 RECIP. NAME: BRUTCHFIELD, D. RECIPIENT AFFILIATION: Operating Reactors Branch 5

SUBJECT: Proposes mod to commitment made in NUREG-0821 per SEP Topic III-5.8 to protect main steam safety & relief valves from effects of block wall failures. Mod would ensure structural integrity.

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ACCESSION NUMBER: 830750443 REC. DATE: 83NOV29
FACIL: 50-SAB Robert Emmet Finn Nuclear Plant, Unit 1, Rochester, N.Y.
AUTHOR: MAIER, J.F. Rochester Gas & Electric Corp.
AUTHOR AFFILIATION: RECIP: NAME RECIP: AFFILIATION
CRUTCHFIELD, D. Operating Reactors Branch 2

SUBJECT: Proposes mod to commitment made in NUREG-0851 per SFP Topic 111-2,8 to protect main steam safety & relief valves from effects of block wall failures. Mod would ensure structural integrity.

TITLE: OR submitted: SFP Topic
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NOTES: NRRDLVSRB 1cy.

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NOTES:



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JOHN E. MAIER
Vice President

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July 20, 1983

Director of Nuclear Reactor Regulation
Attention: Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: SEP Topic III-5.B, Pipe Break Outside Containment
(IPSAR Section 3.3.1.1)
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Crutchfield:

As noted in Section 3.3.1.1 of NUREG-0821, the R. E. Ginna Integrated Plant Safety Assessment Report, December 1982, RG&E had committed to protect the main steam safety and relief valves from the effects of block wall failure. RG&E has made a preliminary investigation of the consequences of such an occurrence, and proposes a modification to our commitment. Our analysis indicates that damage to the main steam safety and relief valves would not prevent safe shutdown, as long as the main steam isolation valves remained operable, and auxiliary feedwater flow could be maintained to the steam generators. In such an event, the total break area would be approximately 2 ft², which is substantially smaller than the design basis steam line break area of 4.37 ft². Thus, reactor coolant system pressure, temperature, and reactivity responses are considered to be enveloped. Auxiliary feedwater would be provided by the Standby Auxiliary Feedwater System (operator action time of 10 minutes is assumed). Other emergency functions, such as Safety Injection System actuation, would be unaffected by damage to the Intermediate Building. Auxiliary feedwater injection, with relief through the openings in the steam lines, would continue until the RHR system could be placed into operation, at which time normal cooldown to cold shutdown could commence.

In order to ensure safe shutdown capability in the event of the block wall failure in the Intermediate Building, RG&E thus proposes the following actions:

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DATE July 20, 1983

TO Mr. Dennis M. Crutchfield

- 1) ensure that the main steam lines and feedwater lines would not lose their structural integrity
- 2) protect the main steam isolation valves, and accessories, as needed, to ensure operation,
- 3) protect the normal motor-driven and turbine-driven auxiliary feedwater connections to the main feedwater lines, up to and including the check valves. This will ensure that standby auxiliary feedwater, which connects to the feedwater lines inside containment, would be routed to the steam generators.

RG&E proposes to perform the necessary analyses and modifications in conjunction with our Structural Upgrade Program, which is presently being reviewed by the NRC.

Very truly yours,



John E. Maier

