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ZWOLINSKI,J.A. Operating Reactors Branch 5

SUBJECT: Provides proposed schedule for meeting requirements of ATWS rule & brief description of tentative design being considered,per Generic Ltr 85-06.Util will use design presented in Section 4.0 of WCAP-10858.

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October 11, 1985

Director of Nuclear Reactor Regulation
Attention: Mr. John A. Zwolinski, Chief
Operating Reactors Branch No. 5
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: ATWS Rule 10 CFR 50.62
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Zwolinski:

The Commission has published 10 CFR 50.62 regarding the reduction of risk from anticipated transients without scram (ATWS) events at light-water cooled nuclear power plants. Section 50.62(d) requires each licensee to develop and submit a proposed schedule for meeting the requirements of the rule within 180 days after the issuance of the QA guidance for non-safety related components. NRC Staff QA guidance was issued April 16, 1985 by Generic Letter 85-06. The purpose of this letter is to provide a proposed schedule for meeting the requirements of the ATWS rule and to provide a brief description of the tentative design being considered to meet the requirements of the rule as it applies to Ginna.

Section 50.62(c)(1) of the rule requires each pressurized water reactor (PWR) to have equipment from sensor output to final actuation device, that is diverse from the reactor trip system, to automatically initiate the auxiliary feedwater system and to initiate a turbine trip under conditions indicative of an ATWS. In a letter from L. D. Butterfield to C. O. Thomas dated July 25, 1985 (OG-156), the Westinghouse Owners Group (WOG) submitted Topical Report WCAP-10858, "AMSAC Generic Design Package", which describes three conceptual designs that meet this requirement at Westinghouse-designed PWRs. RG&E is a member of the WOG and participated in the development of the Topical Report.

RG&E tentatively proposes to use a design similar to that presented in Section 4.0 of WCAP-10858 (Logic 2: AMSAC Actuation on Low Main Feedwater Flow). Turbine trip and auxiliary feedwater flow will be initiated on appropriate indication of low main feedwater flow. The logic will sense conditions indicative of an ATWS event with loss of heat sink, but will not actuate until after the reactor protection system should have completed its safety action.

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Design and procurement for the system will be scheduled to provide for installation during the 1988 refueling outage. This schedule is contingent upon timely Staff review and acceptance of the logic presented in WCAP 10858 and upon completion of other major modifications required by the NRC Staff at Ginna Station in the 1986-1987 time period. Major projects include SPDS installation, Appendix R modifications, reactor vessel level indication system, detailed Control Room Design Review modifications, structural upgrade and other issues.

Very truly yours,

Roger W. Kober

Roger W. Kober