

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8510210050. DOC. DATE: 85/10/11 NOTARIZED: NO DOCKET #  
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244  
 AUTH. NAME: AUTHOR AFFILIATION  
 KOBER, R. W. Rochester Gas & Electric Corp.  
 RECIP. NAME: RECIPIENT AFFILIATION  
 ZWOLINSKI, J. A. Operating Reactors Branch 5

SUBJECT: Provides proposed schedule for meeting requirements of ATWS rule & brief description of tentative design being considered, per Generic Ltr 85-06. Util will use design presented in Section 4.0 of WCAP-10858.

DISTRIBUTION CODE: A055D COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2  
 TITLE: OR/Licensing Submittal; Salem ATWS Events GL-83-28

NOTES: NRR/DL/SEP 1cy.  
 OL: 09/19/69

05000244

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	NRR ORBS BC 01	3		
INTERNAL:	ACRS	6	ADM/LFMB	1
	ELD/HDS4	1	IE/DI	1
	IE/DQAVT	1	NRR/DE/EQB	1
	NRR/DE/MEB	1	NRR/DHFS/HFEB	1
	NRR/DHFS/LQB	1	NRR/DL DIR	1
	NRR/DL/ORAB	1	NRR/DL/SSPB	1
	NRR/DL/TAPMG	1	NRR/DSI/ASB	1
	NRR/DSI/ICSB	2	NRR/DSI/PSB	1
	NRR/DSI/RSB	1	REG FILE	04
	RGN1	1		
EXTERNAL:	24X	1	LPDR	03
	NRC PDR 02	1	NSIC	05
NOTES:		1		

TOTAL NUMBER OF COPIES REQUIRED: LTTR 33 ENCL 0

Faint, illegible text at the top of the page, possibly a header or introductory paragraph.

-----S

① 1 ...

... ..

... ..

... ..

... ..

... ..

... ..

... ..

... ..

... ..

... ..

... ..



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER  
VICE PRESIDENT  
ELECTRIC & STEAM PRODUCTION

TELEPHONE  
AREA CODE 716 546-2700

October 11, 1985

Director of Nuclear Reactor Regulation  
Attention: Mr. John A. Zwolinski, Chief  
Operating Reactors Branch No. 5  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: ATWS Rule 10 CFR 50.62  
R. E. Ginna Nuclear Power Plant  
Docket No. 50-244

Dear Mr. Zwolinski:

The Commission has published 10 CFR 50.62 regarding the reduction of risk from anticipated transients without scram (ATWS) events at light-water cooled nuclear power plants. Section 50.62(d) requires each licensee to develop and submit a proposed schedule for meeting the requirements of the rule within 180 days after the issuance of the QA guidance for non-safety related components. NRC Staff QA guidance was issued April 16, 1985 by Generic Letter 85-06. The purpose of this letter is to provide a proposed schedule for meeting the requirements of the ATWS rule and to provide a brief description of the tentative design being considered to meet the requirements of the rule as it applies to Ginna.

Section 50.62(c)(1) of the rule requires each pressurized water reactor (PWR) to have equipment from sensor output to final actuation device, that is diverse from the reactor trip system, to automatically initiate the auxiliary feedwater system and to initiate a turbine trip under conditions indicative of an ATWS. In a letter from L. D. Butterfield to C. O. Thomas dated July 25, 1985 (OG-156), the Westinghouse Owners Group (WOG) submitted Topical Report WCAP-10858, "AMSAC Generic Design Package", which describes three conceptual designs that meet this requirement at Westinghouse-designed PWRs. RG&E is a member of the WOG and participated in the development of the Topical Report.

RG&E tentatively proposes to use a design similar to that presented in Section 4.0 of WCAP-10858 (Logic 2: AMSAC Actuation on Low Main Feedwater Flow). Turbine trip and auxiliary feedwater flow will be initiated on appropriate indication of low main feedwater flow. The logic will sense conditions indicative of an ATWS event with loss of heat sink, but will not actuate until after the reactor protection system should have completed its safety action.

8510210050 851011  
PDR ADOCK 05000244  
P PDR

A055  
/10

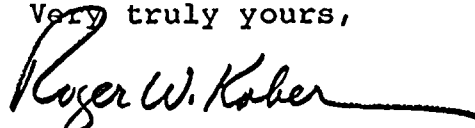


ROCHESTER GAS AND ELECTRIC CORP.  
DATE October 11, 1985  
TO Mr. John A. Zwolinski

SHEET NO. 2

Design and procurement for the system will be scheduled to provide for installation during the 1988 refueling outage. This schedule is contingent upon timely Staff review and acceptance of the logic presented in WCAP 10858 and upon completion of other major modifications required by the NRC Staff at Ginna Station in the 1986-1987 time period. Major projects include SPDS installation, Appendix R modifications, reactor vessel level indication system, detailed Control Room Design Review modifications, structural upgrade and other issues.

Very truly yours,



Roger W. Kober



[Faint, illegible text spanning the top half of the page, possibly bleed-through from the reverse side.]

[Extremely faint and illegible text covering the majority of the page, likely bleed-through from the reverse side.]