



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 25, 1985

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter of December 3, 1984, Rochester Gas and Electric (RG&E) Corporation, the licensee for R. E. Ginna Nuclear Power Plant, submitted a request to amend Facility Operating License No. DPR-18 by changing the Technical Specifications to relax the restriction on movement or stationing of the trolley of the auxiliary building crane over storage racks containing spent fuel.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on May 21, 1985 (50 FR 20987). No public comments or requests for hearing were received.

2.0 EVALUATION

The staff has reviewed the proposed change to Technical Specification 3.11.3 and the licensee's supporting safety evaluation. In the December 3, 1984 letter, the licensee indicated that the trolley of the auxiliary building crane will only carry non-heavy (light) loads (those of weight less than or equal to a spent fuel assembly and its handling tool) over the spent fuel pool. Further, the licensee has stated that the following restrictions will be included in the plant load handling procedures: 1) the rack will not contain spent fuel that has decayed less than 60 days and 2) the current crane interlocks which prevent the crane from travelling over the pool can be defeated only on the approval of the shift supervisor.

Based on our review of the proposed change to Ginna Technical Specification 3.11.3 the staff has determined that it is in accordance with the acceptance criteria of Standard Review Plan section 9.1.4 and is therefore acceptable.

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3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

A. Singh and C. Miller prepared this Safety Evaluation.

Dated: June 25, 1985