



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated December 19, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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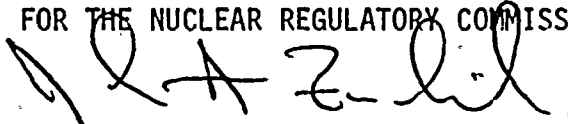
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 5, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 19, 1985.

ATTACHMENT TO LICENSE AMENDMENT NO. 5

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

4.2-1
4.2-2

INSERT

4.2-1
4.2-2

4.2

Inservice Inspection

Applicability

Applies to the inservice inspection of Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator Tubes. It also applies to inservice pump and valve testing.

Objectives

To provide assurance of the continuing structural and operational integrity of the structures, components and systems in accordance with the requirements of 10 CFR 50.55a(g).

Specification

4.2.1

The inservice inspection program for Quality Groups A, B and C components, High Energy Piping Outside of Containment and Steam Generator tubes shall be in accordance with Appendix B of the Ginna Station Quality Assurance Manual. The inservice pump and valve testing program shall be in accordance with Appendix C of the Ginna Station Quality Assurance Manual. These inservice inspection programs shall define the specific requirements of the edition and Addenda of the ASME Boiler and Pressure Vessel Code, Section XI, which are applicable for the forty month period of the ten year inspection interval. The programs' ten year inspection intervals shall be based on the following commencing dates.

4.2.1.1

The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.

4.2.1.2

The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990 and 2000, respectively.

4.2.1.3

The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990.

and 2000, respectively. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

- 4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.
- 4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
- 4.2.1.6 The inspection interval for the Inservice Pump and Valve Testing Program shall be ten year intervals commencing with January 1, 1981, 1990 and 2000.

Basis:

The inservice inspection programs provide assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The programs comply with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures, components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirements of General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.