



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

In a letter dated January 25, 1985, Rochester Gas and Electric Corporation proposed an amendment to Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. The amendment involves a change to Technical Specification 3.8.1(a) (refueling) which would authorize the use of a temporary, specially designed closure plate (with sealed penetrations) in place of the equipment hatch (equipment door) during refueling. The equipment hatch is described in Section 3.8.1.5.4 of the R. E. Ginna updated FSAR. The temporary closure plate is required to facilitate steam generator and other outage maintenance activities while maintaining primary containment integrity during refueling operations.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on February 5, 1985 (50 FR 5020). No public comments or requests for hearing were received.

2.0 EVALUATION

The R. E. Ginna Nuclear Power Plant includes an equipment door approximately fourteen (14) feet in diameter, to permit transfer of large components into and out of the containment. It is constructed of welded steel and has a double-gasketed flange and bolted dished door and is designed to withstand design basis accident pressure. Technical Specification 3.8 specifies the required operating limitations during refueling operations. These requirements ensure that the release of radioactivity within the containment will be restricted from leakage to the environment. The radioactive material released from a postulated fuel handling accident would be retained within the building due to the lack of containment pressurization potential while in the refueling mode coupled with the penetration integrity requirements.

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The specially fabricated temporary closure plate proposed to be utilized during refueling outages will contain sealed penetrations for temporary services and a personnel door that will provide emergency egress. It will be seismically designed, but will not be designed to withstand high pressure. The temporary closure plate will perform the required functions, i.e., provide the required margin of safety for a fuel handling accident by restricting direct communication with the environment and provide a seismic envelope to restrict the potential escape of radioactivity resulting from a postulated seismic event during refueling. The closure plate will conform to the guidance of the NRC Standard Technical Specifications for refueling operations (Section 3/4.9.4) which require that the equipment door be held in place by a minimum of four (4) bolts and that penetrations providing direct access from the containment atmosphere to the outside atmosphere be closed or capable of automatic closure. The intent is that the direct air flow to the environment be restricted but that the containment need not be in a condition to mitigate design basis accidents.

Running temporary service lines through the sealed penetrations will replace the current practice of running them through an open personnel door within the equipment door or attaching a special closure to the personnel door with appropriately sealed penetrations. This will improve personnel safety.

Based on the staff's review, the staff concludes that the licensee's proposal will provide adequate containment integrity during refueling operations. Therefore, the staff finds the proposed modification and Technical Specification change acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

Charles L. Miller prepared this Safety Evaluation.

Dated: March 8, 1985