



**Commonwealth Edison**  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

February 22, 1980

BBS LTR #80-147

James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Reportable Occurrence Report #80-8/03L-0, Docket #050-237, is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

*B. B. Stephenson for 2/25/80*  
B. B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BS:lbg

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

*1002  
3/1*  
FEB 28 1980

# LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 | I | L | D | R | S | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | \_\_\_\_\_ | 5  
8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

1 | L | 0 | 5 | 0 | 0 | 0 | 2 | 3 | 7 | 0 | 1 | 2 | 6 | 8 | 0 | 8 | 0 | 2 | 2 | 5 | 8 | 0 | 9  
8 REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

2 | During routine surveillance, Equipment Operator discovered level in the 2/3 diesel  
3 | generator coolant water expansion head tank overflowing thru the vent line. 2/3  
4 | D/G was declared inoperable. Both Unit 2 and Unit 3 D/G's, low pressure core cooling  
5 | and containment cooling subsystems were proven operable, therefore this event was of  
6 | minimal safety significance. First event of this type at Dresden.  
7 | \_\_\_\_\_  
8 | \_\_\_\_\_

9 | SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE  
E E (11) E (12) D (13) H T E X C H (14) C (15) Z (16)  
9 10 11 12 13 18 19 20  
17 | LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.  
8 0 0 0 8 0 3 L 0  
21 22 23 24 26 27 28 29 30 31 32  
18 | ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NRPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
B Z (18) Z (19) Z (20) Z (21) 0 0 0 0 N (23) N (24) N (25) Y 0 2 1 (26)  
33 34 35 36 37 40 41 42 43 44 47

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

0 | Upon investigation it was determined that the D/G cooling water heat exchanger had  
1 | some tube leaks causing service water to enter the coolant system. Tube leaks will  
2 | occur in heat exchangers due to normal environmental wear. The heat ex-  
3 | changer was pressure tested and seven tubes were plugged. Following repairs the 2/3  
4 | D/G was run satisfactorily. No further action is required.

5 | FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)  
E (28) 0 9 7 (29) Unit 3 in refuel B (31) Daily Diesel Check  
8 9 10 12 13 44 45 46 80

6 | ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
Z (33) Z (34) NA NA  
8 9 10 11 44 45 80

7 | PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)  
0 0 0 (37) Z (38) NA  
8 9 11 12 13 30

8 | PERSONNEL INJURIES NUMBER DESCRIPTION (41)  
0 0 0 (40) NA  
8 9 11 12 30

9 | LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)  
Z (42) NA  
8 9 10 30

0 | PUBLICITY ISSUED DESCRIPTION (45)  
N (44) NA  
8 9 10 30

NRC USE ONLY

NAME OF PREPARER M. Korchynsky

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