



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

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October 23, 1980

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DJS LTR #80-200

James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Revised Reportable Occurrence Report #79-017-01X2, Docket #050-237 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(c), observed inadequacies in the implementation of administrative procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems. This revision corrects errors in Licensee Event Report items 17, 23 and 38.

D. J. Scott  
Station Superintendent  
Dresden Nuclear Power Station

DJS/lg

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
U. S. NRC, Document Mgt. Branch  
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ATTACHMENT TO LICENSEE EVENT REPORT #79-017-01X2  
COMMONWEALTH EDISON COMPANY (CWE)  
DRESDEN UNIT #2  
DOCKET # 050-237

Measured leakage of volume bounded by containment isolation valves A0-2-1601-23, 24, 60, 61, 62 & 63 was 1660 SCFH. This exceeds the T.S. limits for single containment isolation valve allowable leakage, total for testable penetrations and isolation valves and maximum allowable containment leak rate. The leakage was into the secondary containment and the Standby Gas System which resulted in no danger to public health and safety. Similar events were reported by 50-237/76-10, 50-249/76-16 and 50-249/80-07.

Leakage was initially identified to be from the shaft seals on A0-2-1601-60. Subsequent testing indicated leakage on the 1601-23 and 1601-24 valve shaft seals and seat leakage on valve 1601-24. The 1601-24 valve was replaced and shaft seals on valves 1601-23 and 1601-60 were repaired. No further action deemed necessary.

## LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 57 CAT 58

CONT

0 1 REPORT SOURCE L 0 5 0 0 0 2 3 7 7 0 3 2 7 7 9 8 0 9 0 3 8 0 9  
7 8 60 61 68 69 74 75 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Measured leakage of volume bounded by containment isolation valves AO-2-1601-23, 24, 60  
0 3 61, 62 & 63 was 1660 SCFH. This exceeds the T.S. limits for single containment isola-  
0 4 tion valve allowable leakage, total for testable penetrations and isolation valves and  
0 5 maximum allowable containment leak rate. The leakage was into the secondary contain-  
0 6 ment and the Standby Gas System which resulted in no danger to public health and safety.  
0 7 Similar events were reported by 50-237/76-10, 50-249/76-16 and 50-249/80-07.

0 8 7 8 9 80

0 9 SYSTEM CAUSE CAUSE COMPONENT COMP. VALVE  
CODE CODE SUBCODE CODE SUBCODE SUBCODE  
S D 11 E 12 X 13 V A L V E X 14 B 15 D 16  
9 10 11 12 13 18 19 20  
17 LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION  
NUMBER NO. REPORT NO. CODE TYPE NO.  
7 9 0 1 7 0 1 X 2  
21 22 24 26 27 28 29 30 31 32  
ACTION FUTURE EFFECT SHUTDOWN HOURS ATTACHMENT NPRD-4 PRIME COMP. COMPONENT  
TAKEN ACTION ON PLANT METHOD NO. SUBMITTED FORM SUB. SUPPLIER MANUFACTURER  
X 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 Y 24 N 25 P 3 4 0 26  
33 34 35 36 37 40 41 42 43 44 47

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Leakage was initially identified to be from the shaft seals on AO-2-1601-60. Subse-  
1 1 quent testing indicated leakage on the 1601-23 and 1601-24 valve shaft seals and seat  
1 2 leakage on valve 1601-24. The 1601-24 valve was replaced and shaft seals on valves  
1 3 1601-23 and 1601-60 were repaired. No further action deemed necessary.

1 4 7 8 9 80

1 5 FACILITY % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION  
STATUS 0 0 0 N/A B Local Leak Rate Testing  
7 8 9 10 12 13 44 45 46 80

1 6 ACTIVITY CONTENT RELEADED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE  
Z 33 Z 34 N/A N/A  
7 8 9 10 11 12 13 44 45 80

1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION  
0 0 0 37 Z N/A  
7 8 9 10 11 12 13 80

1 8 PERSONNEL INJURIES NUMBER DESCRIPTION  
0 0 0 40 N/A  
7 8 9 10 11 12 80

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION  
Z 42 N/A  
7 8 9 10 80

2 0 PUBLICITY ISSUED DESCRIPTION  
N 44 N/A  
7 8 9 10 80

NRC USE ONLY.

NAME OF PREPARED

John Dunbar

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