

UNITED STATES
ATOMIC ENERGY COMMISSION
DIRECTORATE OF REGULATORY OPERATIONS
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

TELEPHONE
(312) 858-2660

May 12, 1972

Commonwealth Edison Company
ATTN: Mr. Byron Lee, Jr.
Assistant to the President
P. O. Box 767
Chicago, Illinois 60690

Docket No. 50-249

Gentlemen:

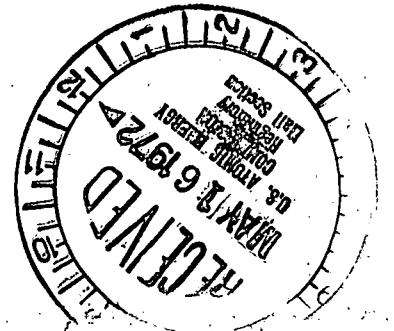
This refers to the inspection conducted by Messrs. Dance and Fiorelli of this office on May 4 and 5, 1972, of the events associated with the May 4, 1972, Dresden Unit 3 reactor scram and safety valve operation, and to discussions of our findings held by Messrs. Dance and Fiorelli with Messrs. Hoyt and Worden of your staff at the conclusion of the inspection.

Areas examined during the inspection included equipment responses to the reactor scram and transient, reactor operating parameters prior and subsequent to the event, consequence of main steam line safety valve operation on components within the drywell, actions taken to correct the failure of the isolation condenser valve to operate and pre-startup and future testing programs initiated to determine the cause of safety valve operation at steam pressure apparently below intended relieving set points. Within these areas, the inspection consisted of selective examination of procedures and representative records, interviews with plant personnel, and observations by the inspectors.

No items of noncompliance with AEC requirements were identified within the scope of this inspection.

It is our understanding, based on discussions with your management representatives, that a program to determine and remedy the cause for the operation of steam line safety valves at lower than intended relief setting is being expedited on a priority basis. We understand that the program will include the following:

1. Obtaining equipment that will allow testing of valves with steam at nominal operating conditions.
2. Evaluation of valve design by testing safety valves in a full scale steam system installation.



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3. Installation and use of appropriate instrumentation on the main steam system of one of your reactor facilities to evaluate system dynamic conditions and the effect of these conditions on safety valve operation.

We will examine your efforts in carrying out this program during subsequent inspections.

Should you have any questions concerning this inspection, we will be glad to discuss them with you.

Sincerely yours,

Boyce H. Grier
Regional Director

cc: W. Worden, Plant Superintendent

bcc: J. G. Keppler, RO
R. H. Engelken, RO
A. Giambusso, L
P. A. Morris, RO
RO Files
DR Central Files
PDR
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