

October 31, 1973

Re: Indian Point Unit No. 2
Facility Operating License
DPR-26
A.O. 3-2-15

Mr. James P. O'Reilly, Director
Regulatory Operations, Region I
U. S. Atomic Energy Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly,

In accordance with the requirements of Technical Specification 6.12.2 of Facility Operating License DPR-26, the following report is provided:

During the course of routine plant status inspections on October 30, 1973, leaks were observed at the 3/4 inch pipe to socket weld branch connections for vent valves S-47 and S-51 associated with the Residual Heat Removal (RHR) System. At the time of these observations, the reactor was depressurized and in the cold shutdown condition.

These venting provisions are not required for operation. Since leaks of a similar nature have occurred before on both of these connections, we plan to eliminate the pipe and valve assemblies at these particular points to eliminate any vibration induced cyclic stress which might have contributed to the failures. The vent assemblies will be replaced with socket welded plugs in the half couplings welded to the RHR System. A survey of similar connections on the RHR System has been made and no other leaks were observed.

An investigation, including, to the extent feasible, metallurgical examinations, is underway. The results of this investigation and final corrective action will be provided in a follow-up report.

Mr. Anthony Fasano of your office was apprised of the circumstances of this occurrence by Mr. John M. Makepeace, by telephone on October 30, and October 31, 1973.

Very truly yours,



Warren R. Cobean, Jr.
Manager, Nuclear Power
Generation

cc: John F. O'Leary