(7.7?) Update Report-Previous LICENSEE EVENT REPORT Report Date 6-30-78 1.4 REQUIRED INFORMATION ](1) (PLEASE PRINT OR TYPE CONTROL BLOCK: I L D R S 3 ] (5) 0 1 LICENSE NUMBER LICENSEE CODE CON'T SOURCE L 6 0 5 0 0 2 4 9 7 0 6 0 7 7 18'3 0'8 18 7 8 75 REPORT DATE 80 0 1 EVENT DATE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) While performing LPCI and CCSW surveillance on Unit 3 prior to taking the U-3 D/G 0 2 out of service, a spike was found on the liquid process radiation monitor for the 0 3 service water discharge. The sample from 3B-1503 CCHX was found contaminated 0 1 It was immediately isolated. Since the other redundant containment cooling heat exchanger 0 5 loop was operable safe plant operation was not impaired. This event occurred twice 0 6 0 7 previously on 2A CCHX. (Reportable occurrences #50-237/77-20 and 77-56) 0 3 9 SYSTEM CODE CAUSE CAUSE COMP VALVE SUBCODE SUBCODE CODE SUBCODE COMPONENT CODE (12) (13) H S F (11)E D T = X = (14)(15) 0 9 [Z (16)С 10 12 13 18 19 REVISION SEQUENTIAL OCCURRENCE REFORT EVENT YEAR REPORT NO. CODE LER/RO TYPE NO. (17 REPORT 0 2 5 8 1.3 0 X 1 NUMBER 20 71 37 ACTION FUTURE SHUTDOWN METHOD NPRD-4 ATTACHMENT SUBMITTED PRIME COMP. COMPONENT FFECT HOURS (22) FORM SUB SUPPLIER MANUEACTURER (21) 0 0 0 0 P 1 <u>X (18) X</u> Z (20) Z Y (23) N (24) N (25) 6 0 (19 (26) 36 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Contaminated sample taken at 3B CCHX service water side indicated possible tube leak. 1:10 Hydro test indicated that one tube was leaking. It was subsequently plugged. 111 1 2 1 3 14 9 8 80 FACILITY METHOD OF OTHER STATUS % POWER DISCOVERY DESCRIPTION (32) 0 7 4 (29) 5 <u>E</u> (23) B (31) NA Spike on process rad monitor 10 12 44 13 45 46 80 ACTIVITY CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) RELEASED\_OF RELEASE ] (33 [M] (34 4.9 X 10-4 curies 6 LL Reactor building service water to 10 11 80 PERSONNEL EXPOSURES: DESCRIPTION (39) NUMBER TYPE 7 0 0 0 37 **Z** (38 PERSONNEL INJURIES 60 DESCRIPTION (41) NUMBER 0 (40) 0 0 NA 11 12 LOSS OF OR DAMAGE TO FACILITY 'έυ (43) DESCRIPTION 9 <u>Z</u> (42) NA 10 80 PUBLICITY DESCRIPTION (45) NRC USE ONLY ISSUED 44 0 I N NA 1 10 63 ū9. 30 John Achterberg 265 NAME OF PREPARER PHONE:

## ATTACHMENT TO LICENSEE EVENT REPORT 78-025/03X-1 COMMONWEALTH EDISON COMPANY (CWE) DRESDEN UNIT-3 (ILDRS-3) DOCKET # 050-249

While performing LPCI and CCSW surveillance on Unit 3 prior to taking the Unit 3 D/G out of service, a spike was found on the liquid process radiation monitor for the service water discharge. A sample taken from the 3B-1503 CCHX was found to be contaminated. A tube leak was suspected, and the heat exchanger was immediately isolated. At no time during the duration of the LPCI and CCSW surveillance was liquid radioactive waste discharged to the river from Unit 2&3 or Unit 1 radwaste systems. Since the other redundant containment cooling heat exchanger loop was operable safe plant operation was not impaired.

Hydro test results revealed that 1 of the 2510 tubes in the heat exchanger was leaking. Calculations to estimate the amount of radioactive material released were performed using the following parameters. The service water volume trapped in the CCSW piping, when the pumps are not running, is 4,340 gallons. The length of each release was conservatively estimated at 1.24 minutes. The circulating water flow was about 1,017,000 gpm. The activity of the water on the service water side was conservatively assumed to be the same as the activity of the torus water. A gamma isotopic analysis of the D-3 torus water revealed the concentrations of the following nuclides.

Mn Co Co Cs Cs	54 58 60 134 137	1.2 X 9.0 X 2.2 X 1.5 X 4.2 X	$10^{-6}$ $10^{-7}$ $10^{-5}$ $10^{-6}$ $10^{-6}$ $10^{-7}$	uCi/ml uCi/ml uCi/ml uCi/ml uCi/ml
لات Zn	65	4.2 X 6.0 X	$10^{-7}$	uCi/ml uCi/ml

Calculations based on the activity of the contaminated service water and the available dilution flow, revealed that the sum of the ratios of these isotopic concentrations to the maximum permissible concentration was 0.003. This indicates that the concentration in the circulating water canal to the Dresden cooling lake was 0.3% of the applicable 10CFR20 limit. This radioactivity was further diluted by 1275 acres of water in the cooling lake. Thus the health and safety of the public was not affected.

The faulty tube was plugged on both the top and bottom with 3/4" stainless steel tapered plugs. A post maintainence hydro was maintained at about 110 psig for an hour with no other leaks observed. The heat exchanger is a type 6B-3222 heat exchanger manufactured by Berlin Chapman, a division of Perfex Corporation and was built to ASME III, class "C" and the Tubular Exchanger Manufacturer's Association class "R" standards.