



**Commonwealth Edison**  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

*D. Lenham*  
FILE

April 21, 1978

BBS LTR #758-78

James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Reportable Occurrence Report #78-015/03L-0, Docket #050-249 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation.

B. B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BBS:cac

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

APR 25 1978

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | I | I | D | R | S | 3 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

CON'T 01 | L | 0 | 5 | 0 | 0 | 0 | 2 | 4 | 9 | 7 | 0 | 3 | 3 | 0 | 7 | 8 | 8 | 0 | 4 | 2 | 1 | 7 | 8 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 | During required inspection to check 10% of all hydraulic snubbers during refueling,
03 | one snubber in drywell was functionally inoperable by bench test. Safety implications
04 | were minimal because there have been no seismic events requiring the shock arresting
05 | capability of the failed snubbers and redundant ECCS systems were available.

09 | S | F | E | X | S | U | P | O | R | T | D | Z | 17 | 7 | 8 | 0 | 1 | 5 | 0 | 3 | L | 0 | C | F | Z | Z | 0 | 0 | 0 | 0 | Y | Y | X | B | 2 | 1 | 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 | Failures resulted from heat and radiation which promote fluid leakage from seal deter-
11 | ioriation and possible change in fluid viscosity. This operability test is in addition
12 | to the accessible and inaccessible snubber inspections performed earlier and provides
13 | additional means of verifying operability. Snubber was subsequently replaced with
14 | Mechanical snubber.

15 | H | 0 | 0 | 0 | NA | B | Observation during test

16 | Z | Z | NA | NA

17 | 0 | 0 | 2 | E | 100 MR-Contractors

18 | 0 | 0 | 0 | NA

19 | Z | NA

20 | N | NA

ATTACHMENT TO LICENSEE EVENT REPORT 78-015/03L-0  
COMMONWEALTH EDISON COMPANY (CWE)  
DRESDEN UNIT 3 (ILDRS3)  
DOCKET #050-249

In accordance with Technical Specification 4.6.I, 10% of the safety-related hydraulic snubbers were functionally tested during refueling outage to verify their operability. These tests were performed in addition to and separate from the prior inaccessible and accessible snubber inspections and were performed to further verify reliability of snubbers. Five (5) snubbers were removed for testing--Three (3) from the drywell, one (1) from the torus and one (1) from the isolation condenser pipeway. Of these five, one snubber (#15) from the drywell failed to perform as intended. As required by Tech Specs a work request was then initiated to remove an additional 10% for operability testing as required by Technical Specification. These additional five (5) were subsequently bench tested for operability and all performed satisfactorily.

The characteristics of failure for this snubber are the same as explained in Licensee Event Report #78-10-01T-0. This report described the reasons for the failure of six (6) snubbers that were tested under the "Inaccessible Snubber Inspection" requirements.

The snubber was for seismic protection of LPCI Line 1506-16", which connects into the "B" recirc loop for injection into the reactor vessel. Safety implications were minimal because there has been no need for seismic protection during this operating cycle and redundant ECCS systems were available.