

April 13, 1988

Docket No. 50-244

Mr. Roger W. Kober, Vice President  
Electric & Steam Production  
Rochester Gas & Electric Corporation  
89 East Avenue  
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: SNUBBER REPLACEMENT PROGRAM AT GINNA

On November 20, 1987, a summary report of the Steam Generator Snubber Replacement Program was provided to the staff for review. This report described a proposed modification to the existing SG upper lateral support configuration and an analysis to demonstrate the acceptability of resulting loads from postulated seismic and other design bases events. The intent of the proposed modification is to replace six of the eight hydraulic snubbers of each SG with rigid structural members (bumpers).

Subsequent discussions between RGE personnel and NRC staff have identified the need for additional information in order for us to complete the review. It was also determined that your request for such a modification should be submitted as an amendment to the Technical Specification. Enclosed is the request for additional information on the snubber replacement program.

To resolve the remaining issues as expeditiously as possible, please provide your proposed amendment no later than 30 days from the receipt of this letter. If a meeting is required to further clarify the enclosed requests, do not hesitate to call.

Sincerely,  
*Original signed by.*  
Carl Stahle, Project Manager  
Project Directorate I-3  
Division of Reactor Projects I/II

Enclosures:  
As stated

cc: See next page

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Request For Additional Information

on Ginna SG Snubber

Replacement Program

1. The size and basis of the bumper gaps in the cold condition.
2. The detailed calculations of the cold-shut-down condition loads in all steam generator supports, reactor vessel supports and reactor coolant pump supports, when subjected to SSE seismic loading.
3. The calculation of the minimum, maximum and average steam generator upper stiffnesses and their inclusion in the RCL model.
4. The justification of the thrust coefficients used for the time-history analysis of the steam generator outlet nozzle and the feedwater nozzles.
5. Description of the non-linear time-history analyses of the RCL when subjected to loading due to postulated breaks at the pressurizer surge, RHR and SI accumulator nozzles, and the SG steam outlet nozzle and the feedwater nozzles. This should include the specified time-history loading forcing functions.
6. Provide clarification of the modeling and calculational results of the two analyses which are performed in the hot condition.

Mr. Roger W. Kober  
Rochester Gas and Electric Corporation R. E. Ginna Nuclear Power Plant

cc:

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