UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO.19 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated October 9, 1985, the Rochester Gas and Electric Corporation (RG&E or the licensee) submitted a proposed license amendment for Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant (the facility). The amendment deletes the requirement from the Technical Specifications (TS) for operation of the Auxiliary Building ventilation and charcoal filter adsorber system when the fuel being moved or stored in the spent fuel storage pool had decayed at least 60 days since irradiation.

2.0 EVALUATION

The NRC staff has performed a previous evaluation (letter dated November 14, 1984, NRC to RG&E) in regard to an expansion of the capacity of the spent fuel pool. Our evaluation indicated that the radiological consequences of nine damaged fuel assemblies, without operation of the charcoal filters, resulted in an offsite dose of 2 rem thyroid and 0.1 rem whole body. This release is two-thirds of one percent and 0.4 of one percent, respectively, of the accident releases assumed as acceptable in 10 CFR 100.11(a)(2). The licensee's proposed changes to TS 3.11 and 4.11 in the October 9, 1985 submittal deals with the handling of only a single fuel assembly, when the charcoal filters are not in operation and the fuel has decayed for at least 60 days. We have evaluated the licensee's proposal and have determined that operation in the proposed manner meets the requirements of 10 CFR Part 100, 10 CFR Part 20 and 10 CFR Part 50 Appendix I. In addition, the guidance of NUREG-0472 (Radiological Effluent Technical Specifications for PWR's) for gaseous effluent treatment systems would be met in the event of an accident during the proposed mode of operation. The largest offsite dose resulting from our analyses was 2.09 rem thyroid for a single fuel assembly. These evaluations were performed by our contractor, the Idaho National Engineering Laboratory, and submitted to NRC by letter dated August 19, 1986 which transmitted their TER Number EGG-PHY-7350 dated August 1986. We conclude that the changes to TS 3.11 and 4.11 are acceptable as the postulated accidents result in offsite doses that are a small fraction of the acceptable accident consequences of 10 CFR Part 100.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

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Dated: September 18, 1986

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