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 KOBER, R. W. Rochester Gas & Electric Corp.
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 LEAR, G. E. PWR Project Directorate 1

SUBJECT: Advises that mod to insulation on pressurizer relief valve loop seals rescheduled for Spring 1987 outage, due to lead time required for procurement of engineered items. Mod responds to NUREG-0737 Item II. D. 1

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ACCESSION NBR: 880310077 DOC DATE: 8/10/84 NOTARIZED: NO
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 AUTHOR AFFILIATION
 ROBERT R. J. Rochester Gas & Electric Corp.
 RECIPIENT AFFILIATION
 PWR Project Director 1

SUBJECT: Advises that mod to insulation on pressurizer relief valve
 loop seals rescheduled for Spring 1987 outage, due to lead
 time required for procurement of engineered items Mod
 responds to NUREG-0737 Item 11.D.1

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ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700

March 24, 1986

Director of Nuclear Reactor Regulation
Attention: Mr. George E. Lear, Chief
PWR Project Directorate No. 1
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUREG-0737, Item II.D.1 - Relief and Safety Valve
Performance
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Lear:

By letter dated May 24, 1985, Rochester Gas and Electric responded to NRC Staff requests for additional information on testing of relief and safety valves. In our response to Question 11, we indicated that a modification to the insulation on the pressurizer relief valve loop seals would be performed during the Spring, 1986 outage. However, due to the lead time required for procurement of the engineered items, and the scope of other outage activities, this modification was not performed. It has been rescheduled for the Spring 1987 outage.

The continued operation of Ginna prior to completion of these modifications is acceptable for the following reasons. The EPRI tests, upon which the safety valve piping analysis was based, used two widely separated loop seal temperature profiles. The "hot loop seal" test used a minimum temperature of approximately 350°F, with the "cold loop seal" test using essentially ambient temperatures. No tests were conducted at temperatures between those two conditions. While the temperature measurements of the Ginna loop seal were in total inconclusive, a majority of the loop seal volume was measured at a temperature most consistent with the hot loop seal case. As a result, forces generated in downstream piping would also closely represent those in the hot loop seal case used in the Ginna piping analysis. The effect on downstream piping from the small volume of water in the loop seal in which the temperature could not be verified to be "hot" would not be expected to inhibit safety valve operation or compromise primary system integrity.

Very truly yours,

Roger W. Kober
Roger W. Kober

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Distribution:
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 P. Shuttleworth w/Encl.
 Ginna PM w/Encl.

March 18, 1986

DOCKET NO(S). 50-244
 Mr. Roger W. Kober
 Vice President
 Electric and Steam Production
 Rochester Gas & Electric Corporation
 89 East Avenue
 Rochester, New York 14649
 SUBJECT: R. E. GINNA NUCLEAR POWER PLANT

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statment, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____, dated _____.
- Notice of Hearing on Application for Construction Permit, dated _____.
- Notice of Consideration of Issuance of Facility Operating License, dated _____.
- Monthly Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated _____.
- Application and Safety Analysis Report, Volume _____.
- Amendment No. _____ to Application/SAR dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____, dated _____.
- Order Extending Construction Completion Date, dated _____.
- Other (Specify) Bi-weekly Notice covering period January 15, 1986. Expiration date
for hearing requests and comments February 14, 1986.

Division of PWR Licensing-A
 Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

OFFICE➤	PAD#1:LA...
SURNAME➤	P. Shuttleworth
DATE➤	3/18/86.....

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MEMORANDUM FOR THE RECORD

1. SUMMARY

2. PURPOSE

3. DISCUSSION

4. CONCLUSIONS

5. RECOMMENDATIONS

6. ACTION

7. DISTRIBUTION

Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

cc:

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