



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-18  
ROCHESTER GAS AND ELECTRIC CORPORATION  
R. E. GINNA NUCLEAR POWER PLANT  
DOCKET NO. 50-244

INTRODUCTION

1.0 By letters dated March 10, 1987 and January 26, 1988, the licensee, Rochester Gas & Electric Corporation, proposed changes to the Ginna Technical Specifications which would address changes in the surveillance requirements made necessary by the installation of automatic actuation of the shunt trip attachments of the reactor trip breakers as required by Generic Letter (GL) G.L.83-28, Item 4.3. The changes delineated in G.L.83-28 were revised by the issuance of G.L.85-09. The licensee's proposal was reviewed and additional changes were requested to make the Technical Specifications conform to G.L.85-09 requirements. These changes were incorporated by the licensee's submittal of August 26, 1988. On January 17, 1989 the licensee resubmitted the proposed amendment because the format of the table in the Technical Specification required a change, following the acceptance by NRC of a previous amendment.

2.0 EVALUATION

The licensee's Technical Specifications differ in some respects from the Standard Westinghouse Technical Specifications which were used as the basis for the requirements of G.L.85-09. Consequently, the originally proposed changes to address new surveillance test requirements for the shunt trip attachment of the reactor trip breakers did not fully meet the requirements of G.L.85-09.

This proposed amendment constitutes an agreement by the licensee to incorporate monthly staggered testing of the two trains of Reactor Trip Logic and the Reactor Trip Breakers. In addition, the licensee revised Action Statement 14 of Table 3.5-1 and modified footnote 5 of table 3.5-1 to comply with the G.L.85-09 requirements. All other aspects of Generic Letter 83-28, Item 4.3 are resolved to the satisfaction of the staff.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that the amendment involves

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no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published in the Federal Register a proposed finding that this amendment involves no significant hazards consideration, 53 Fed. Reg. 9513 (March 23, 1988). There has been no public comment on such finding. Subsequent licensee submittals of August 26, 1988 and January 17, 1989 were only minor corrections to the original submittal; therefore, it was determined that it was not necessary to renotice the application. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

The staff has evaluated the licensee's request to incorporate on-line reactor trip breaker testing into the Technical Specifications. The change in the test requirements will provide a more reliable reactor trip breaker system.

The staff concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: APR 04 1989

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