



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION  
DOCKET NO. 50-244  
R. E. GINNA NUCLEAR POWER PLANT  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34  
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated March 10, 1987, January 26, 1988, August 26, 1988 and January 17, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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PDR ADOCK 05000244  
P FDC

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 30 days from date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director  
Project Directorate I-3  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: APR 04 1969

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.5-8  
3.5-15  
4.1-7  
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INSERT

3.5-8  
3.5-15  
4.1-7  
4.1-7a\*

\*Indicates new page

TABLE 3.5-1 (Continued)  
PROTECTION SYSTEM INSTRUMENTATION

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> TOTAL NO. of CHANNELS	<u>2</u> NO. of CHANNELS TO TRIP	<u>3</u> MIN. OPERABLE CHANNELS	<u>4</u> PERMISSIBLE BYPASS CONDITIONS	<u>5</u> OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 3 CANNOT BE MET	<u>6</u> CHANNEL OPERABLE ABOVE
19.	Degraded Voltage 480V Safeguards Bus	2/bus	2/bus	1/bus		7	T <sub>max</sub> = 350°F
20.	Automatic Trip Logic Including Reactor Trip Breakers	2	1	2	Note 4	14	Note 5

NOTE 1: When block condition exists, maintain normal operation.

NOTE 2: Channels should be operable at all modes below the bypass condition with the reactor trip system breakers in the closed position and control rod drive system capable of rod withdrawal.

NOTE 3: Channels shall be operable at all modes below the bypass condition except during refueling defined to be when fuel is in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

NOTE 4: One reactor trip breaker may be bypassed for surveillance testing provided the other reactor trip breaker is operable.

NOTE 5: Channels shall be operable at all modes above refueling when the control rod drive system is capable of rod withdrawal unless both reactor trip breakers are open.

F.P. = Full Power

12. With the number of operable channels less than the Total Number of Channels, operation may proceed provided the inoperable channel is placed in the tripped condition within 1 hour. Should the next Channel Functional Test require the bypass of an inoperable channel to avoid the generation of an actuation signal, operation may proceed until this Channel Functional Test. At the time of this Channel Functional Test, or if at any time the number of operable channels is less than the Minimum Operable Channels required, be at hot shutdown within 6 hours and at an RCS temperature less than 350°F within 6 hours.
13. With the number of operable channels less than the Minimum Operable Channels required, operation may continue provided the containment purge and exhaust valves are maintained closed.
14. Should one reactor trip breaker or channel of trip logic be inoperable the plant must not be in the operating mode following a six hour time period, and the breaker must be open.

If one of the diverse reactor trip breaker trip features (undervoltage or shunt trip attachment) on one breaker is inoperable, restore it to operable status within 48 hours or declare breaker inoperable. If at the end of the 48 hour period one trip feature is inoperable it must be repaired or the plant must not be in the operating mode, and the reactor trip breaker must be open, following an additional six hour time period. The breaker shall not be bypassed while one of the diverse trip features is inoperable except for the time required for performing maintenance to restore the breaker to operable status.

TABLE 4.1-1 (CONTINUED)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
25. Containment Pressure	S	R	M	Narrow range containment pressure (-3.0, +3 psig) excluded
26. Steam Generator Pressure	S	R	M	
27. Turbine First Stage Pressure	S	R	M	
28. Emergency Plan Radiation Instruments	M	R	M	
29. Environmental Monitors	M	NA	NA	
30. Loss of Voltage/Degraded Voltage 480 Volt Safeguards Bus	NA	R	M	
31. Trip of Main Feedwater Pumps	NA	NA	R	
32. Steam Flow	S	R	M	
33. T <sub>avg</sub>	S	R	M	
34. Chlorine Detector, Control Room Air Intake	NA	R	M	
35. Ammonia Detector, Control Room Air Intake	NA	R	M	
36. Radiation Detectors, Control Room Air Intake	NA	R	M	
37. Reactor Vessel Level Indication System	M	R	NA	
38a. Trip Breaker Logic Channel Testing	NA	NA	M	Notes 1, 2 and 3
38b. Trip Breaker Logic Channel Testing	NA	NA	R	Note 1

Amendment No. 9, 30, 34

4.1-7

TABLE 4.1-1 (CONTINUED)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
39. Reactor Trip Breakers	NA	NA	M	Function test - Includes independent testing of both undervoltage and shunt trip attachment of reactor trip breakers. Each of the two reactor trip breakers will be tested on alternate months.  Note 2
40. Manual Reactor Trip	NA	NA	R	Includes independent testing of both undervoltage and shunt trip circuits. The test shall also verify the operability of the bypass breaker.
41a. Reactor Trip Bypass Breaker	NA	NA	M	Using test switches in the reactor protection rack manually trip the reactor trip bypass breaker using the shunt trip coil.
41b. Reactor Trip Bypass Breaker	NA	NA	R	Automatically trip the undervoltage trip attachment.

NOTE 1: Logic trains will be tested on alternate months corresponding to the reactor trip breaker testing. Monthly logic testing will verify the operability of all sets of reactor trip logic actuating contacts on that train (See Note 3). Refueling shutdown testing will verify the operability of all sets of reactor trip actuating contacts on both trains. In testing, operation of one set of contacts will result in a reactor trip breaker trip; the operation of all other sets of contacts will be verified by the use of indication circuitry.

NOTE 2: Testing shall be performed monthly, unless the reactor trip breakers are open, or shall be performed prior to startup if testing has not been performed within the last 30 days.

NOTE 3: The source range trip logic may be excluded from monthly testing provided it is tested within 30 days prior to startup.

