ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8811300527 DOC.DATE: 88/11/23 NOTARIZED: NO DOCKET #
FACIL:50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
AUTH.NAME AUTHOR AFFILIATION
MECREEDY,R.C. Rochester Gas & Electric Corp.
RECIP.NAME RECIPIENT AFFILIATION
RUSSELL,W.T. Region 1, Ofc of the Director

SUBJECT: Special rept:on 881018,turbine driven auxiliary feedwater pump inoperability.

DISTRIBUTION CODE: IE22D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 2
TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL
	PD1-3 LA	1	1	PD1-3 PD	1	1
	STAHLE,C	1	1			
INTERNAL:	ACRS MICHELSON	1	1	ACRS MOELLER	2	2
	ACRS WYLIE	1	1	AEOD/DOA	1	1
	AEOD/DSP/TPAB	1	1	AEOD/ROAB/DSP	1	1
	ARM/DCTS/DAB	1	1	DEDRO	1	1
	NRR/DEST/ADS 7E	1	0	NRR/DEST/CEB 8H	1	1
	NRR/DEST/ESB 8D	1	1	NRR/DEST/ICSB 7	1	1
	NRR/DEST/MEB 9H	1	1	NRR/DEST/MTB 9H	1	1
	NRR/DEST/PSB 8D	1	1	NRR/DEST/RSB 8E	1	1
	NRR/DEST/SGB 8D	1	1	NRR/DLPQ/HFB 10	1	1
	NRR/DLPQ/QAB 10	1	1	NRR/DOEA/EAB 11	1	1
	NRR/DREP/RAB 10	1	1	NRR/DREP/RPB 10	2	2
	NRR/DRIS/SIB 9A	1	1	NUDOCS-ABSTRACT	1	1
	REG FILE 02	1	1	RES/DSIR/EIB	1	1
	RES/DSR/PRAB	1	1	RGN1 FILE 01	1	1
EXTERNAL:	EG&G WILLIAMS,S	4	4	FORD BLDG HOY,A	1	1
	H ST LOBBY WARD	1	1	LPDR	1	1
	NRC PDR	1	1	NSIC HARRIS,J	1	1
	NSIC MAYS,G	1	1			

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 43 ENCL 42

R I D S / A D D S

R I D S / A D D S

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER N.Y.

November 23, 1988

Mr. William T. Russell, Regional Administrator
U.S. Nuclear Regulatory Commission
Region 1
475 Allendale Road
King of Prussia, PA 19406

Subject: Thirty (30) Day Special Report
Turbine Driven Auxiliary Feedwater Pump Inoperability
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Russell:

Ginna Station's Technical Specification Section 3.4.2b requires that a thirty (30) day special report be submitted if one auxiliary feedwater pump or flow path is inoperable for more than seven days.

On October 18, 1988, the Turbine Driven Auxiliary Feedwater System was declared inoperable per procedure A-52.4, "Control of Limiting Conditions for Operating Equipment". A Nonconformance Report identified that the discharge line check valves had not received reverse closure testing for back leakage as specified by ASME Section XI. In addition, apparent existing check valve leakage was indicated by higher than ambient temperatures in the Turbine Driven Auxiliary Feedwater (TDAFW) discharge piping upstream from the check valves.

Check valves CV-4003 and CV-4004 provide the barrier to prevent feedwater from feeding into the turbine driven auxiliary feedwater pump (TDAFWP) discharge lines from the respective A/B steam generators. They also provide a barrier to prevent cross flow condition from occurring between steam generators in the event of a pressure imbalance. Valves CV-4003 and CV-4004 are three (3) inch 900 pound Crane-Chapman tilting disc check valves installed during original plant construction.

IE22
1/1

Declaring the TDAFW System inoperable was taken as a precautionary measure due to the inability to immediately quantify the existing back leakage. Corrective action is progressing along two parallel paths.

An analysis will be performed to determine acceptable back leakage limits, assuring that ASME Section XI acceptance criteria of "no gross leakage", and accident analysis assumptions are met.

The Maintenance Department is taking steps to ensure valve back leakage is minimized by ordering and installing new seat and disc assemblies or by installing refurbished seat and disc assemblies as removed from the respective valves. A RG&E qualified valve maintenance contractor is performing the refurbishment as well as fabricating a valve seat closure test device for testing seat leakage at a variety of test pressures in order to determine closure integrity over a range of system operating pressures.

The test device will be utilized to assess the "as found" condition of the presently installed seat/disc assemblies.

In the interim, the TDAFW system is isolated with the manual valves MV-4005 and MV-4006, located downstream of the suspect check valves.

Should system operation be required as a result of a blackout condition, Procedure ECA-0.0, "Loss of all A.C.", as written addresses the establishment of auxiliary feedwater flow to the required steam generator(s). During operation of the pump in this condition, the header is charged, and thus, the backflow condition is not a concern.

The turbine driven auxiliary feedwater pump system is anticipated to be made fully operable by December 20, 1988.

Very truly yours,

Robert C. Mecredy
General Manager
Electric Production

xc: U.S. Nuclear Regulatory Commission (Original)
Document Control Desk
Washington, D.C. 20555

xc: Ginna Senior Resident Inspector